

# Reactors and Fuels

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Short Course: Introduction to Nuclear  
Chemistry and Fuel Cycle Separations

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December 16, 2008

# Focus On:

- Characteristics relevant to separation of used nuclear reactor fuel into its constituent parts: nuclear fuel reprocessing
  - Minimal discussion of nuclear reactors per se
- Fuels most relevant to the U.S. and separations
- Civilian nuclear power

# Outline

- Metal clad fuels: oxide and variants
  - Description and fabrication
  - Fuels for many reactors
    - Light-water reactors (LWRs)
      - Boiling Water Reactors (BWRs)
      - Pressurized Water Reactors (PWRs)
    - Fast Reactors cooled with Na, K, Bi, Pb, He
- Graphite-based fuels
  - Description of fuels and how they are fabricated
    - High-Temperature Gas-Cooled Reactors (HTGRs)
      - Also called Very-High-Temperature Reactors (VHTRs)
    - Pebble-Bed Modular (Gas Cooled) Reactor (PBMR)
- Spent fuel characteristics

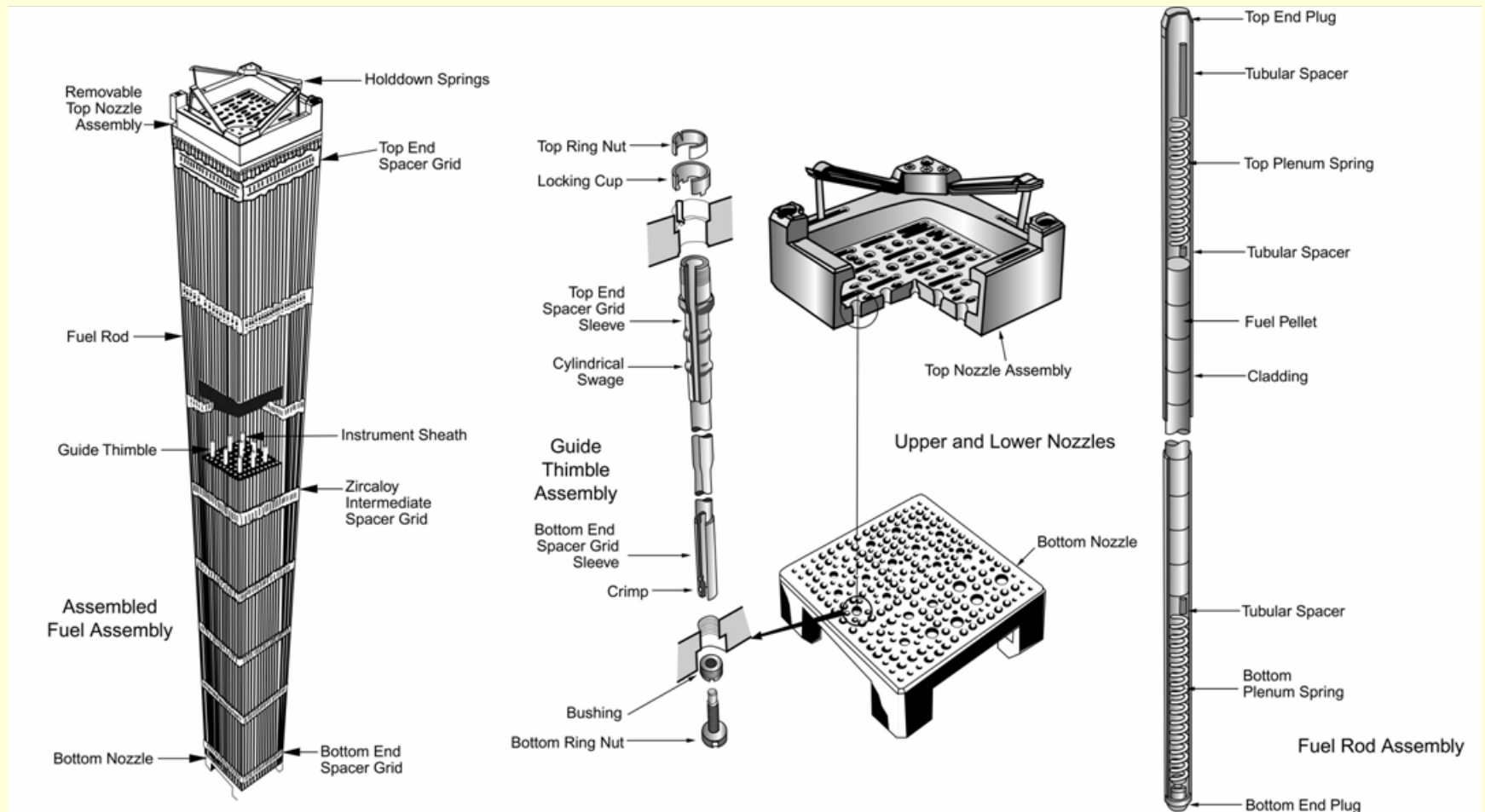
# Metal-Clad Fuels



# Pressurized Water Reactor (PWR)

- Dimensions: square,  $H=4.1\text{m}$ ,  $21\text{cm} \times 21\text{cm}$
- Weight: 460 kgU, 520 kg  $\text{UO}_2$ , 135 kg hardware
  - Hardware mostly Zircaloy (Zr with Sn, Fe, Cr)
  - Grid spacers: Zircaloy, Inconel, stainless steel
  - End pieces: Stainless steel, Inconel
- Fuel element array:  $14 \times 14$  to  $17 \times 17$
- Fuel element size: 1 cm OD,  $H=3.9\text{m}$
- Enrichment: 3-5%
- May have separate burnable poison rods

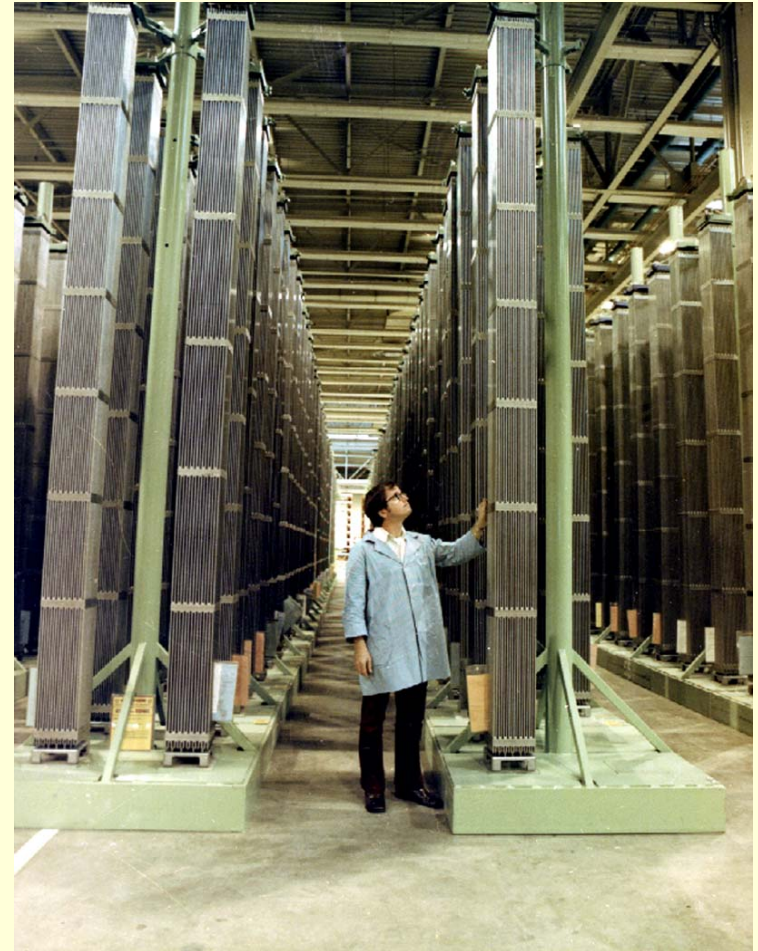
# Pressurized Water Reactor





# Pressurized Water Reactor

Pressurized-Water  
Fuel Assembly





# Boiling Water Reactor (BWR)

- Dimensions: square, H=4.5m, 14 cm x 14 cm
- Weight: 180 kgU, 210 kg UO<sub>2</sub>, 110 kg hardware
  - Hardware mostly Zircaloy (Zr with Sn, Fe, Cr)
  - Grid spacers: Zircaloy
  - Channel (aka shroud): Zircaloy
  - End pieces: Stainless steel
- Fuel element array: 8 x 8
- Fuel element size: 1.25 cm OD, H=4.1m
- Enrichment: 2.5-4.5%
- May have Gd in some rods and variable enrichment in 3-D



# Boiling Water Reactor

## BWR/6 FUEL ASSEMBLIES & CONTROL ROD MODULE

- 1.TOP FUEL GUIDE
- 2.CHANNEL FASTENER
- 3.UPPER TIE PLATE
- 4.EXPANSION SPRING
- 5.LOCKING TAB
- 6.CHANNEL
- 7.CONTROL ROD
- 8.FUEL ROD
- 9.SPACER
- 10.CORE PLATE ASSEMBLY
- 11.LOWER TIE PLATE
- 12.FUEL SUPPORT PIECE
- 13.FUEL PELLETS
- 14.END PLUG
- 15.CHANNEL SPACER
- 16.PLENUM SPRING

GENERAL  ELECTRIC





# PWR and BWR Fuel Variants

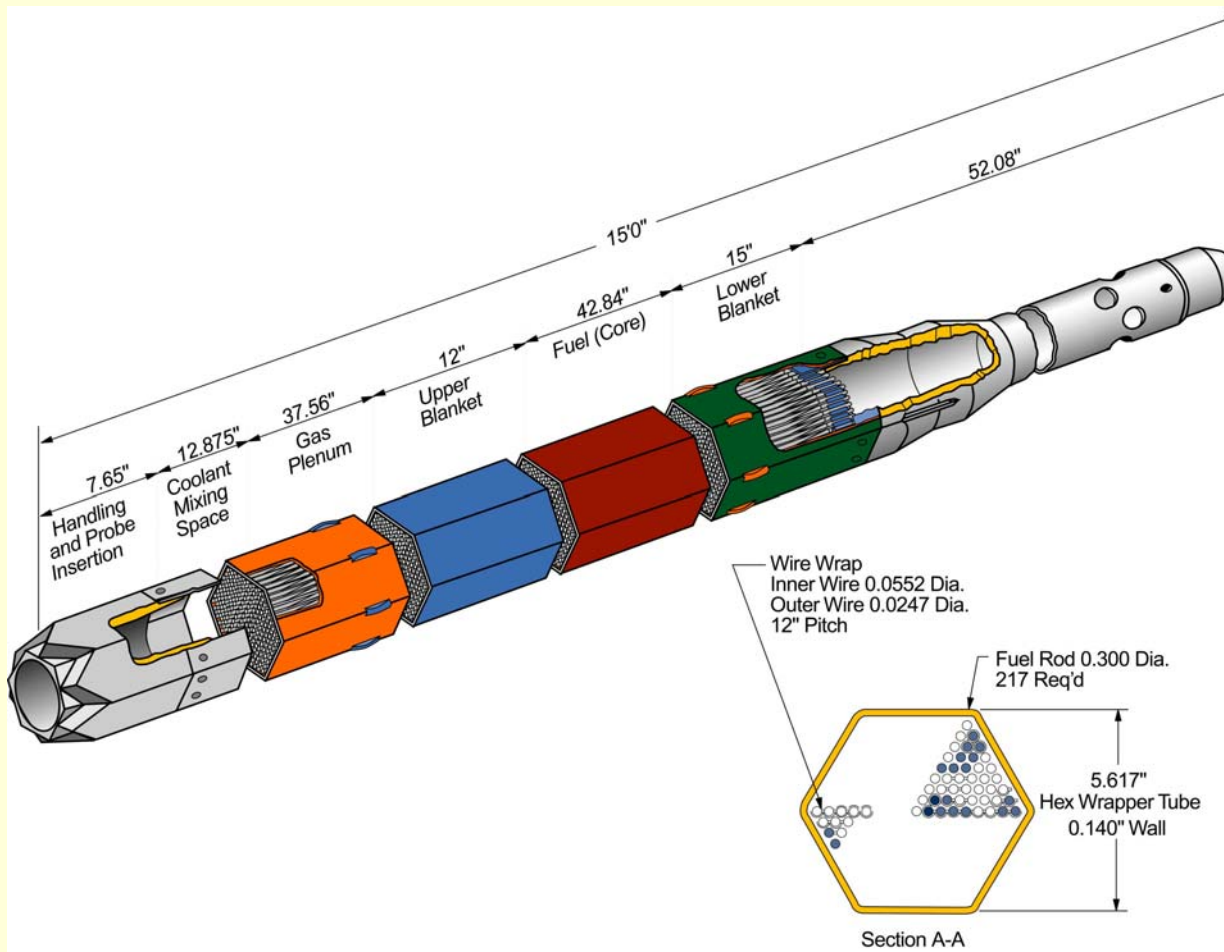
- Advanced Cladding
  - Zirlo (Zr-Nb alloy) and Duplex (layered) for PWRs
  - Tweak Zircaloy composition for BWRs
- Moving to more smaller elements
- Mixed-oxide (MOX) fuel
  - U- 5-8% Pu
    - Already being done in US (weapons Pu) and elsewhere
  - Mixed actinides from advanced reprocessing
    - U-Np-Pu: modest extension of U-Pu fuel technology
    - Am-Cm: a challenge, probably requires targets
- More PWR fuel gradation and burnable poisons
- Thorium oxide fuel



# Fast Breeder Reactor

- Dimensions: hexagonal,  $H=4-5.5\text{m}$ ,  $W$  (flats)= 10-20 cm; HM height  $\sim 2\text{m}$
- Weight:  $\sim 60\text{ kgHM}$ ,  $\sim 65\text{ kg MOX}$ ,  $\sim 135\text{ kg}$  hardware (core plus axial blanket)
  - Hardware: Stainless steel
  - Mostly wire wrap for pin spacing
- Fuel element array: 200-300 pins
- Fuel element size: 0.6-0.9 cm OD,  $H=4-5\text{m}$
- Enrichment: 15-30% Pu
- Blanket: All depleted  $\text{UO}_2$ 
  - Fewer, larger diameter elements

# Fast Reactor Fuel Assembly





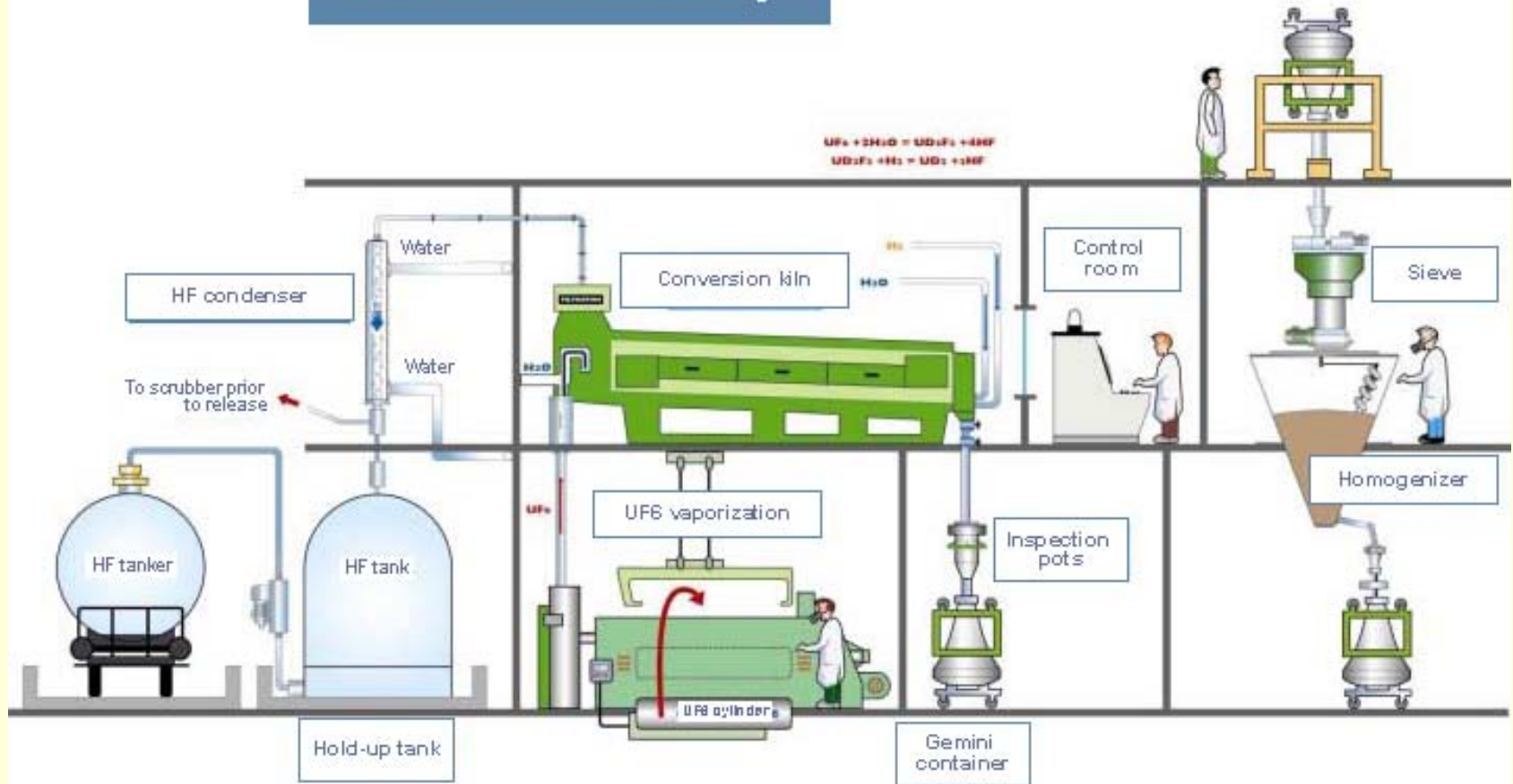
# Fast Reactor Fuel Variants

- Designs not settled: considerable variation in number of elements, dimensions, and weights possible
- Reduce breeding/conversion ratio to achieve net destruction of transuranics
  - Eliminate fertile blankets in favor of non-fertile neutron reflectors (e.g., stainless steel)
  - Inert matrix (e.g.,  $\text{ZrO}_2$ ) fuel
- Carbide, nitride, or metal fuel instead of oxide

# Metal-Clad Fuel Fabrication

# UO<sub>2</sub> Fuel Fabrication

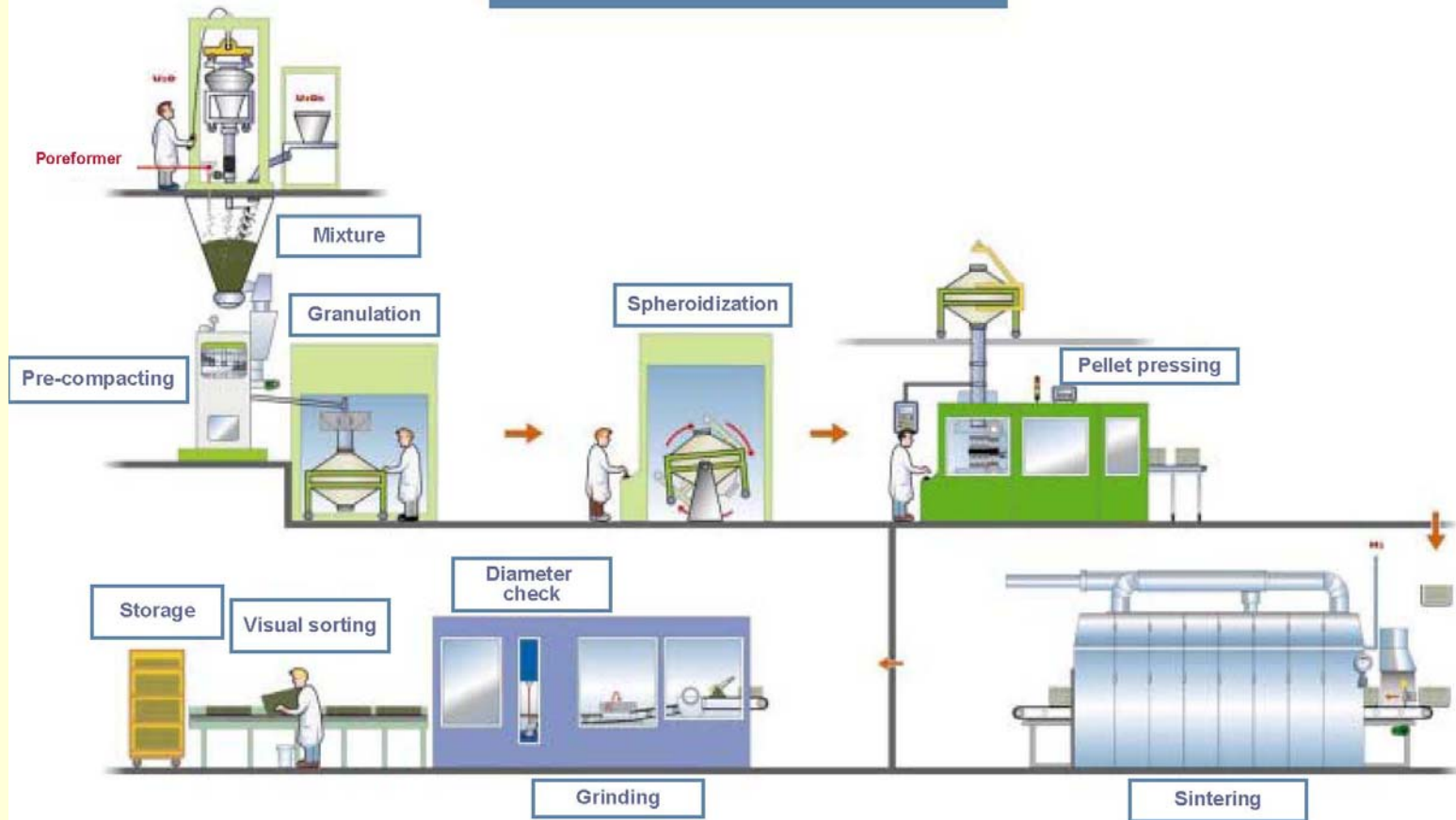
## Conversion facility





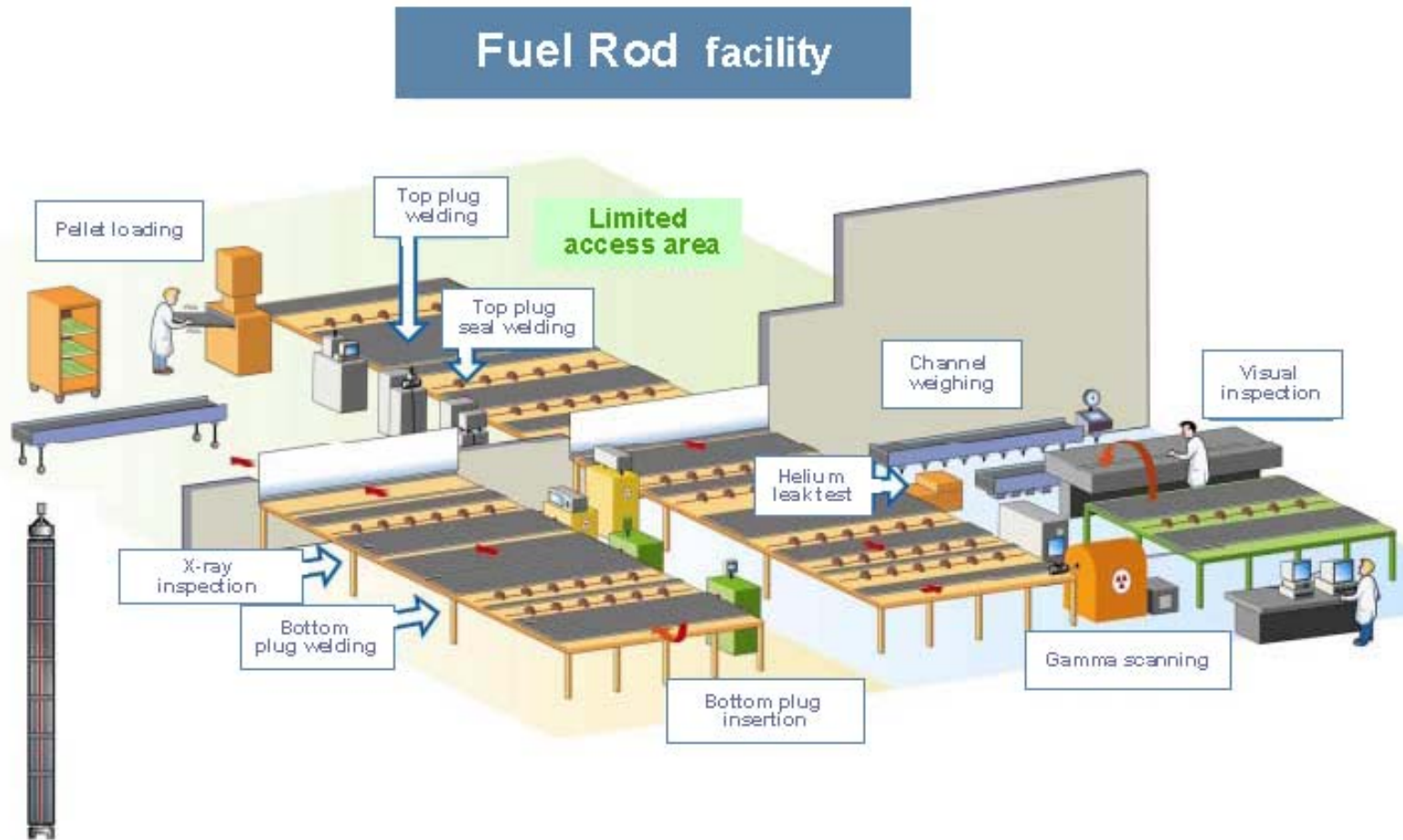
# UO<sub>2</sub> Fuel Fabrication

## Pelletization facility



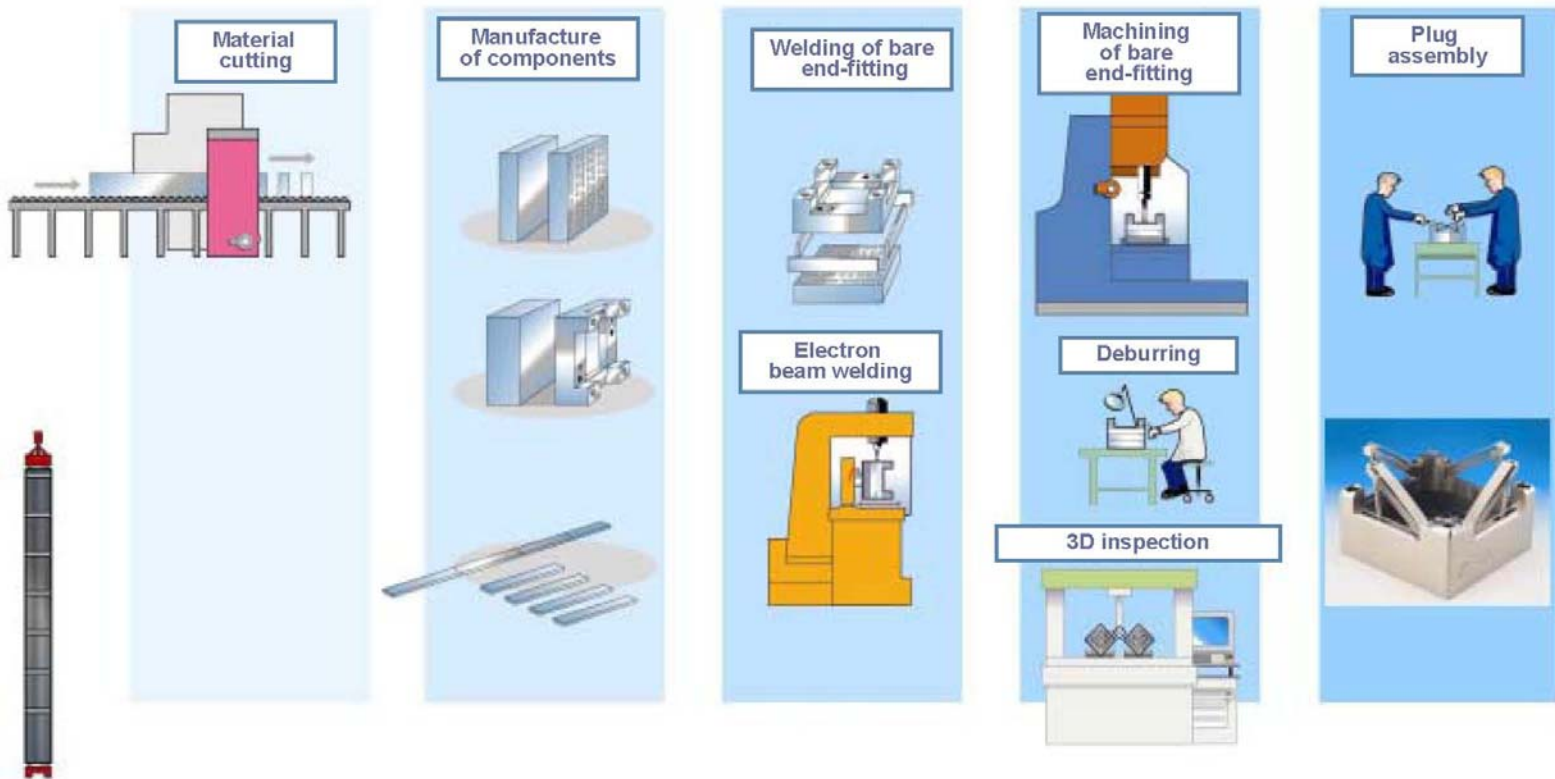


# UO<sub>2</sub> Fuel Fabrication

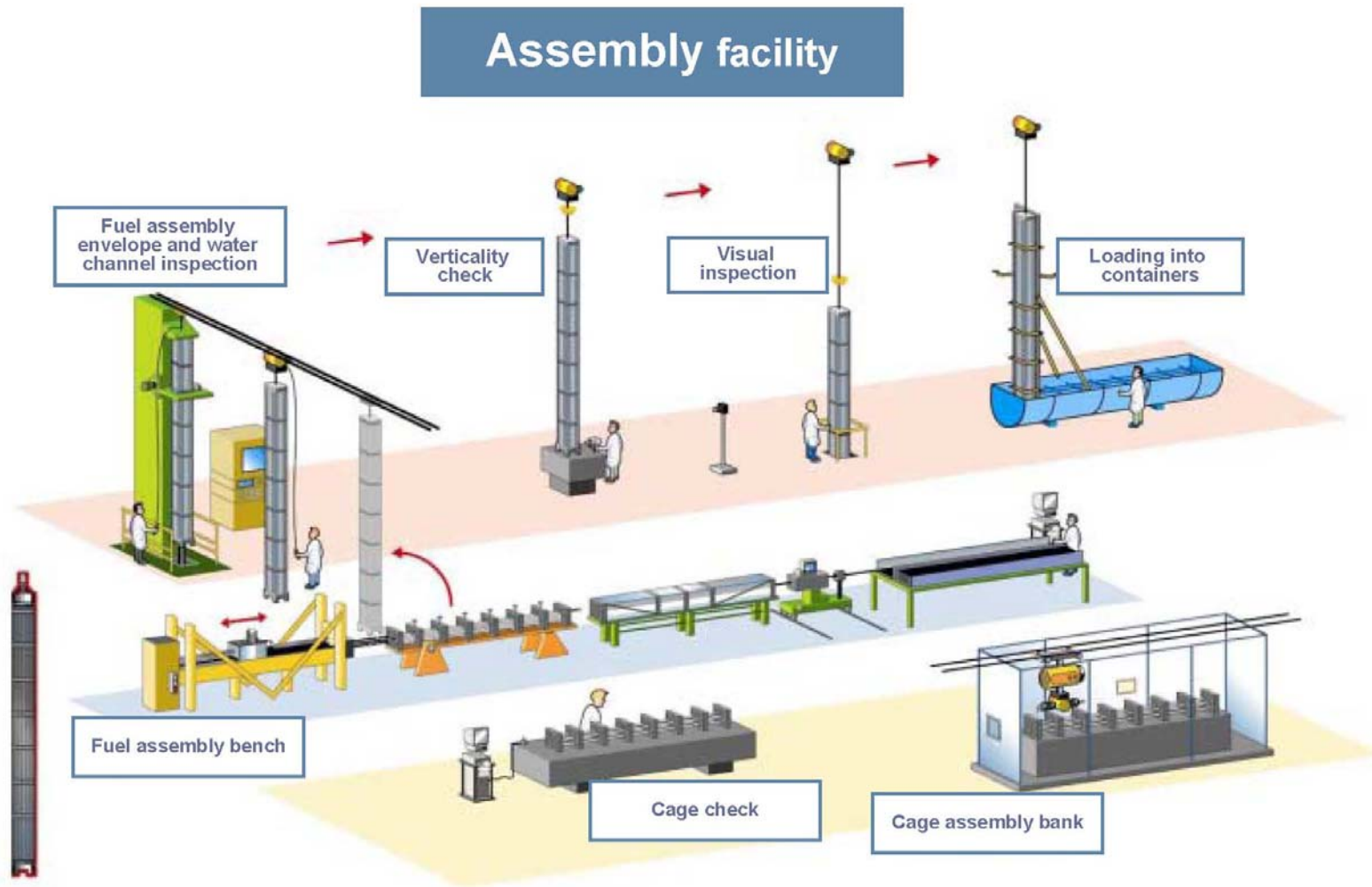


# UO<sub>2</sub> Fuel Fabrication

## Machining facility



# UO<sub>2</sub> Fuel Fabrication



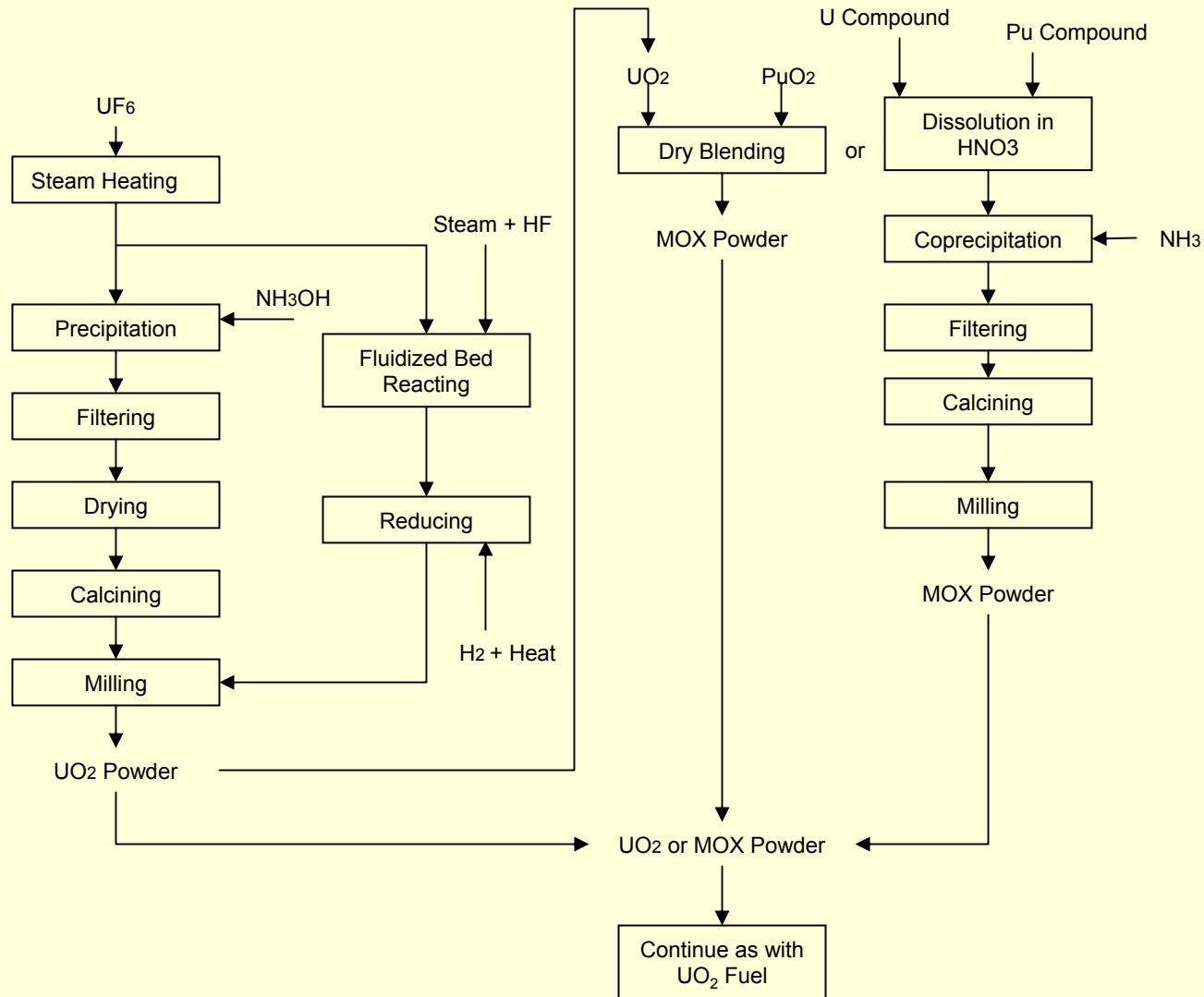


# MOX Fuel Fabrication

- The same as  $\text{UO}_2$  fuel manufacture however....
  - Need to make sure the oxides are very homogenous and have required proportions
    - MOX fuel fabrication from powder usually dilutes a high-concentration “master blend” to the proper Pu enrichment
- Mixed-oxide (MOX): mixtures of actinide oxides
  - Conventionally, U-Pu
  - Advanced: U-Pu-Np, Am-Cm, combinations
- Conventional MOX fuels are being tested in the US but are currently being produced elsewhere
  - Facility using weapons Pu is being built at SRS



# MOX Fuel Fabrication





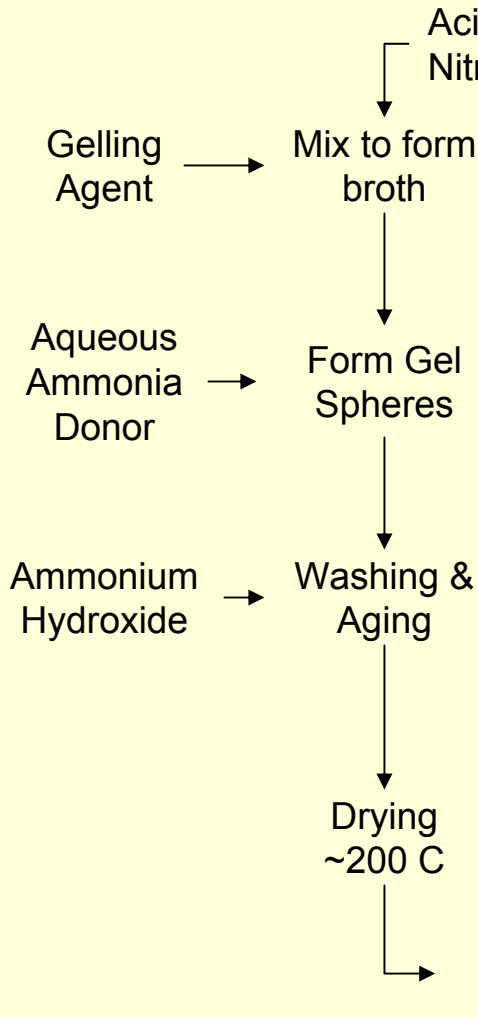
# Sol-Gel/Sphere-Pac Fabrication

- Used to prepare oxide fuels (U and MOX) without mixing powders, grinding, and dust
- Involves two steps:
  - Gelation: form kernels of U, Pu, and/or Th oxides by forming spheres
    - Based on ammonia precipitation of hydrated heavy metal oxides
    - 30, 300, and 1200 $\mu$ m optimal for Sphere-Pac (85% T.D.)
    - External gelation seems to be the current preference
  - Spheres are then washed and dried at  $\sim 200^{\circ}\text{C}$
  - Sphere-Pac: Calcine and then sinter the spheres and load them into clad tubes

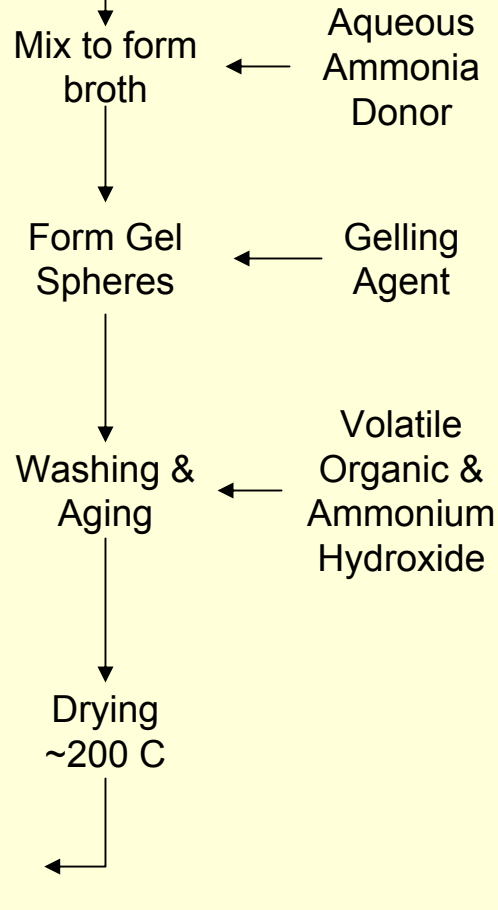


# Sol-Gel/Sphere-Pac Fabrication

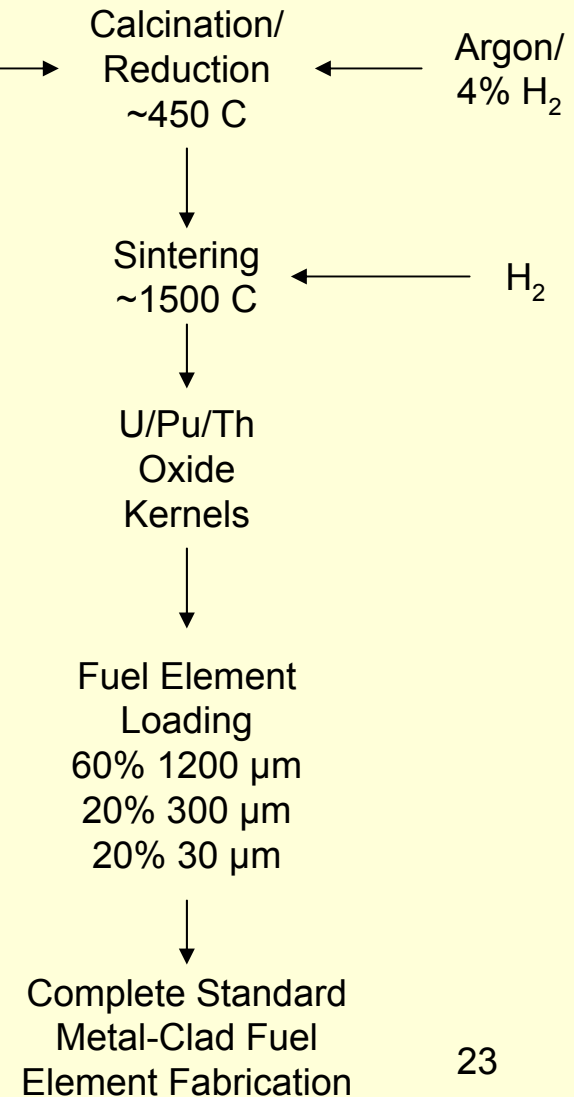
## External Gelation



## Internal Gelation



## Sphere-Pac



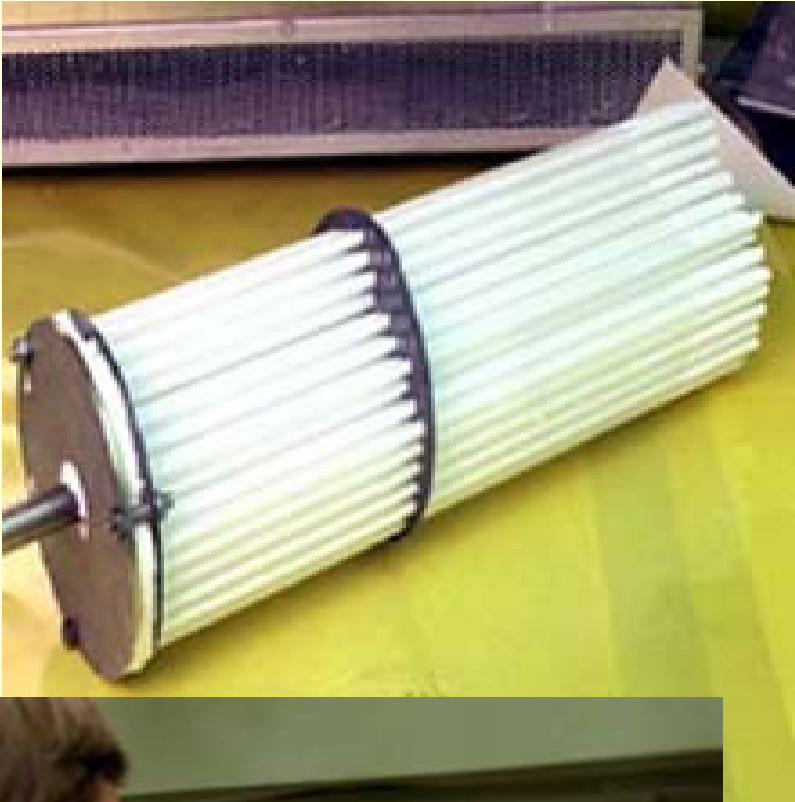


# Metal Fuel Fabrication

- Begin with a furnace containing a molten mixture of actinides, some fission products, and alloying constituents
- Make a mold having an array of quartz tubes
- Insert mold in furnace, seal, and evacuate
- Lower mold into melt and increase pressure to force metal melt into tubes
- Raise mold, cool, break to yield metal fuel “pellets” ~0.5m long.
- Insert in metal clad as with oxide fuels



# Metal Fuel Fabrication





# Fabrication Scrap Recycle

- All fabrication processes internally recycle off-specification fuel
  - Essentially dissolution and U/Pu/MOX purification steps used in a reprocessing plant
  - Off-spec metal fuels simply go back into the melt furnace or electrorefiner

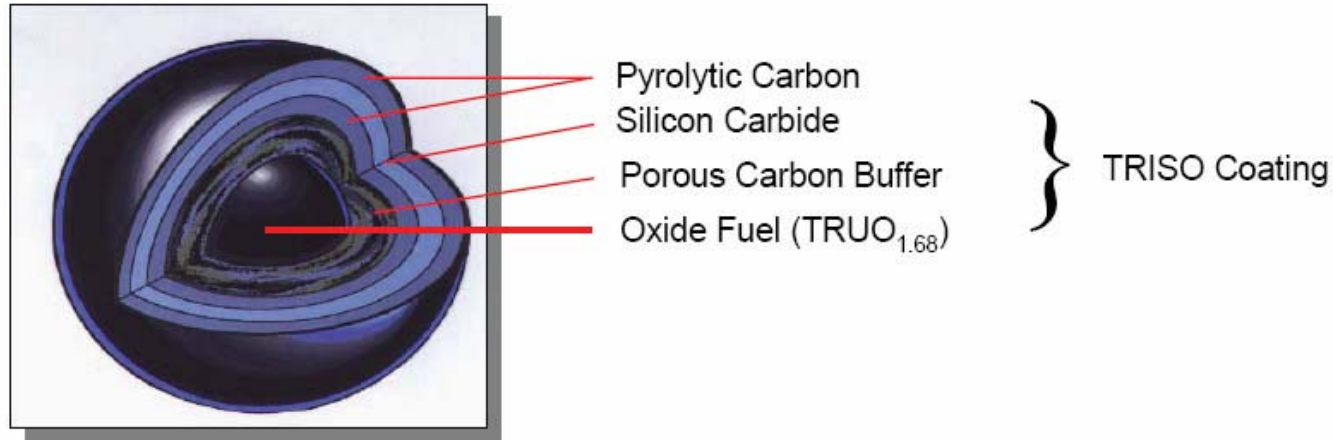
# Graphite-Based Fuels



# HTGR Prismatic Fuel

- Dimensions: hexagonal,  $H=0.8\text{m}$ ,  $0.36\text{m}$  (flats)
- Weight: 5-7 kgU, 5.5-7.5 kg  $\text{UO}_2$ 
  - Hardware 126 kg C (mostly graphite), 4 kg SiC
- ~1000 blocks for 600 MW(t) reactor
- Fuel element array: 210 on a triangular pitch
  - 108 Coolant channels
- Fuel element size: 1.3 cm OD,  $H=0.8\text{m}$ 
  - Contains 14-15 “compacts” with 350-500 $\mu\text{m}$  TRISO particles
- Enrichment: 8-20%
- May have separate  $\text{B}_4\text{C}$  burnable poison rods<sup>28</sup>

# HTGR Prismatic Fuel



**PARTICLES**

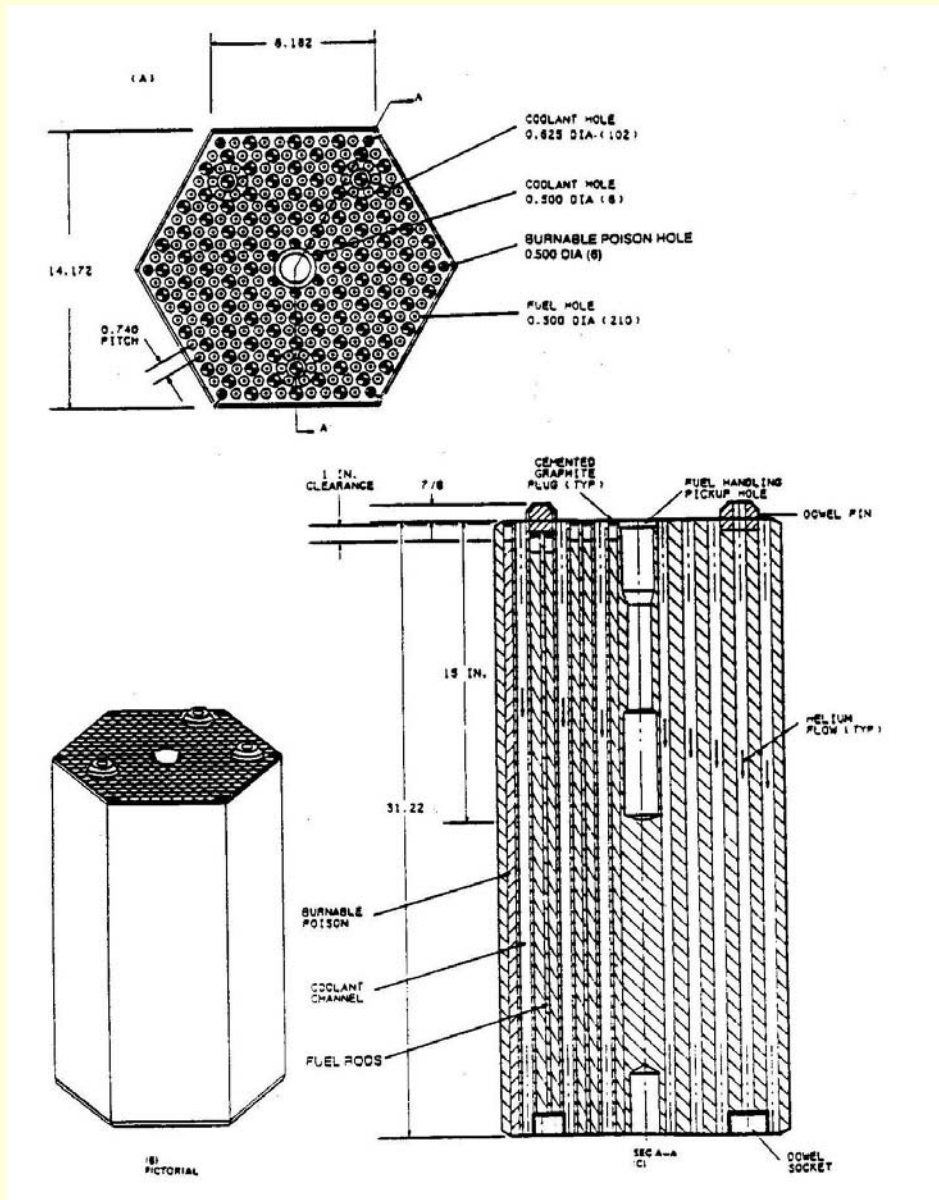


**COMPACTS**



**FUEL ELEMENTS**

# HTGR Prismatic Fuel





# Pebble Bed Reactor Fuel

- Dimensions: Spherical,  $D = 6.0$  cm
- Weight: 9 g U, 10 g  $\text{UO}_2$ 
  - “Hardware” 194 g C (mostly graphite), ~6 g SiC
- ~360,000 pebbles for 400 MW(t) reactor
- Fuel element array: random pile
- Fuel element size:
  - 900 $\mu\text{m}$  TRISO particle
  - ~15,000 particles per pebble
- Enrichment: 7-10%





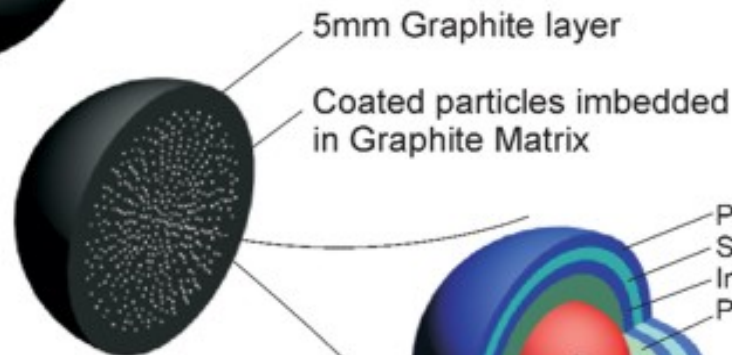
# Pebble Bed Reactor Fuel

## FUEL ELEMENT DESIGN FOR PBMR

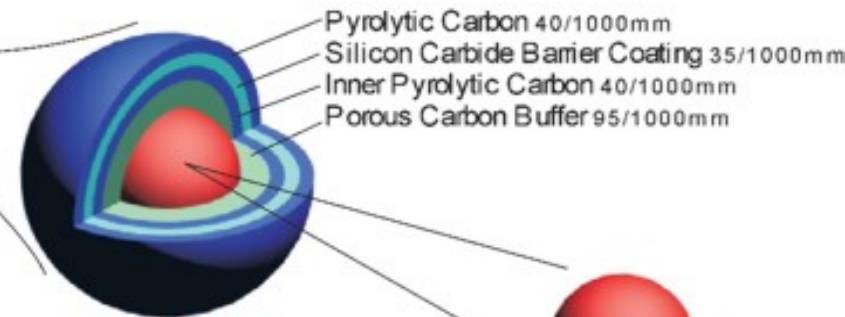


Dia. 60mm

**Fuel Sphere**



**Section**



Dia. 0,92mm

**TRISO**  
**Coated Particle**



Dia. 0,5mm

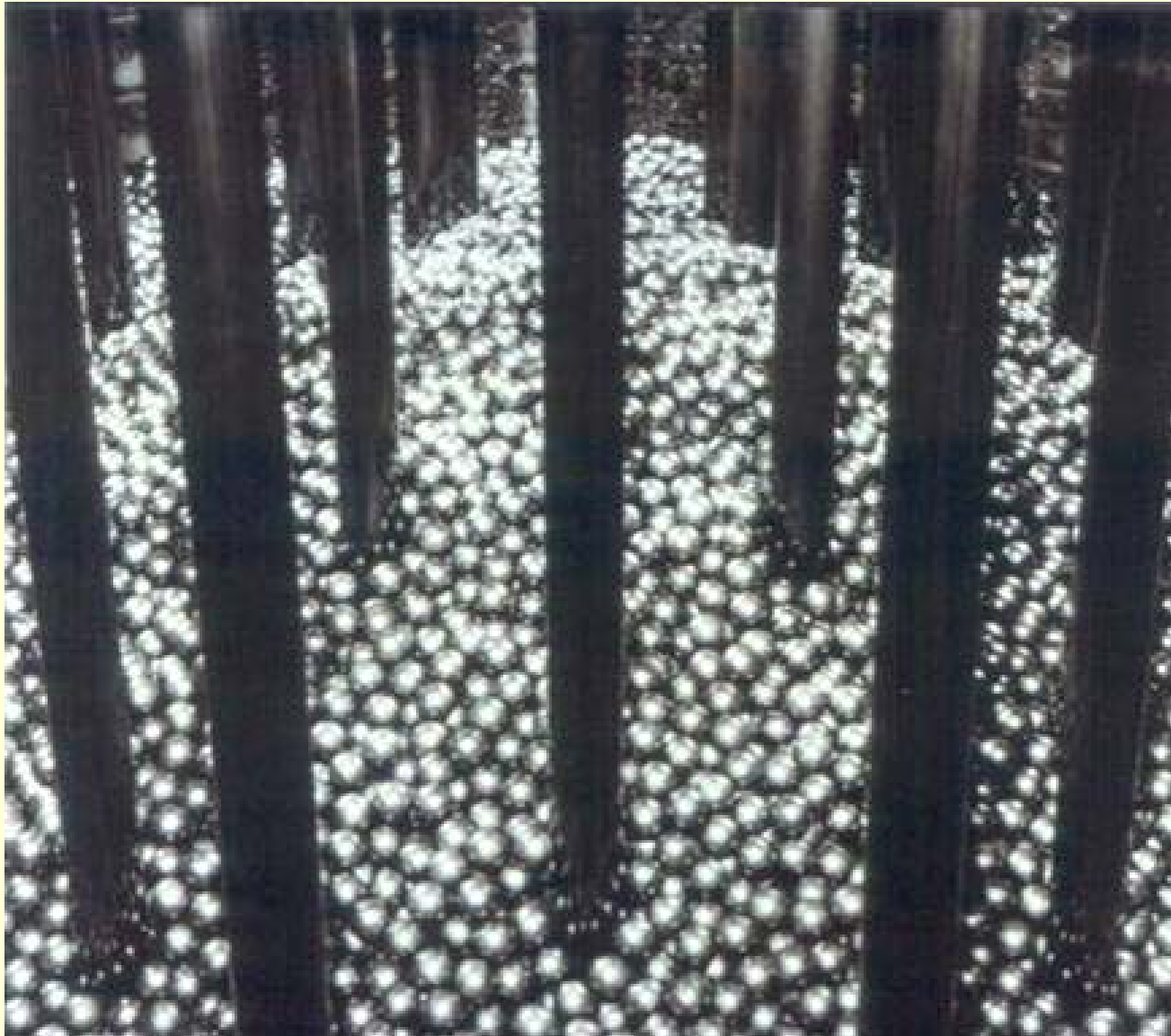
Uranium Dioxide  
**Fuel Kernel**







# Pebble Bed Reactor

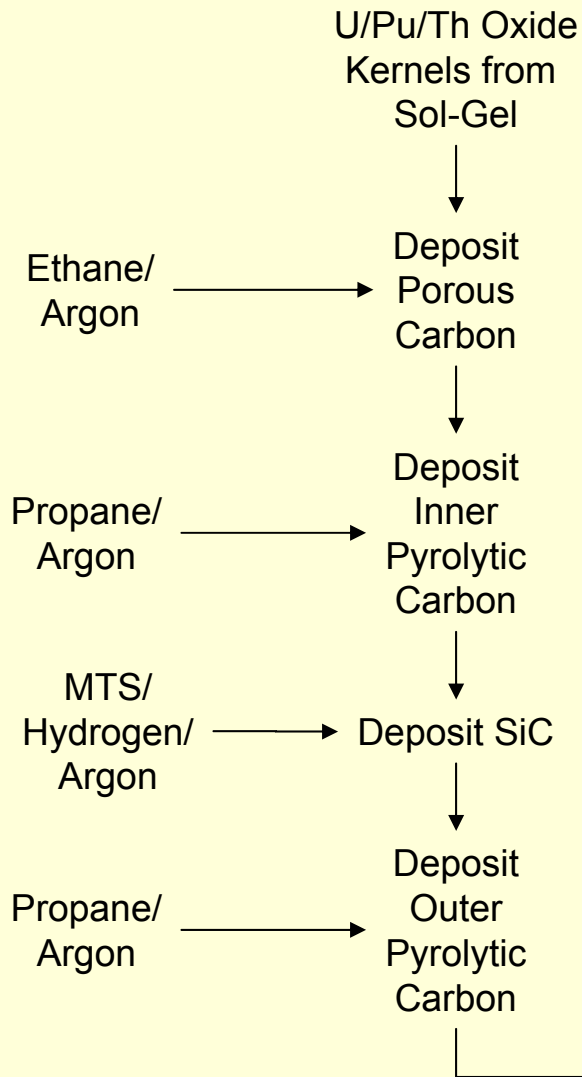


# Graphite Fuel Fabrication

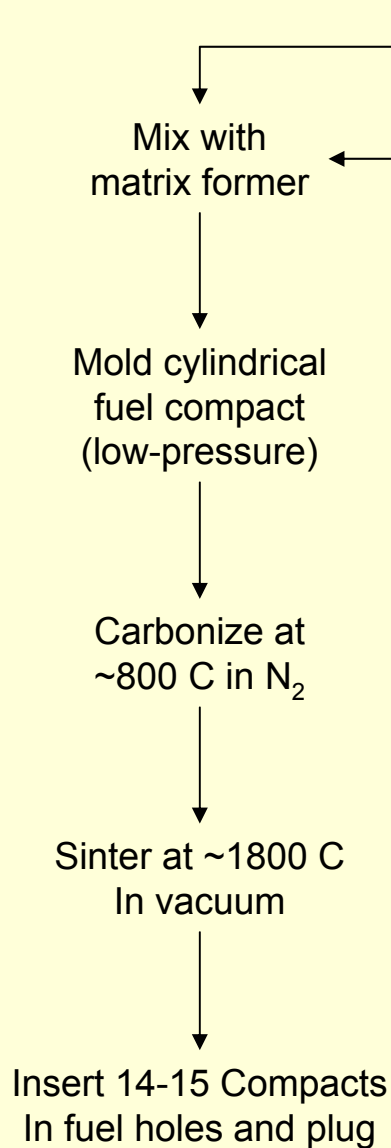


# Graphite Fuel Fabrication

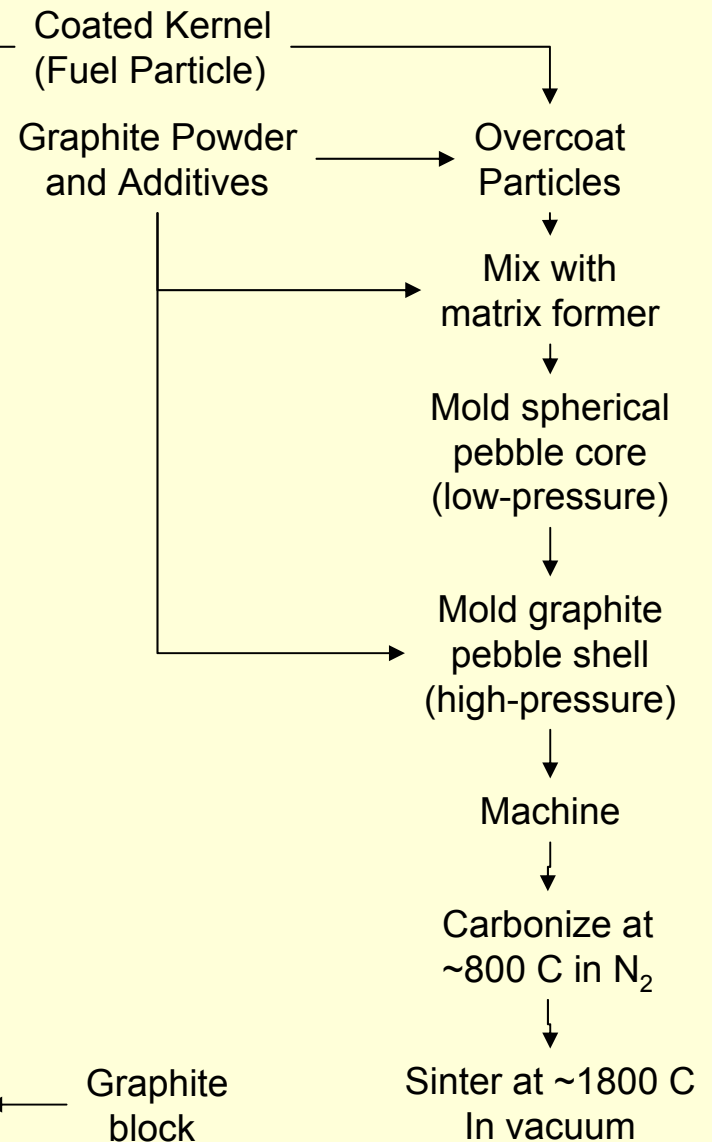
## Kernel Coating



## HTGR Prism Fuel



## PBMR Pebble Fuel



# Spent Fuel Characteristics

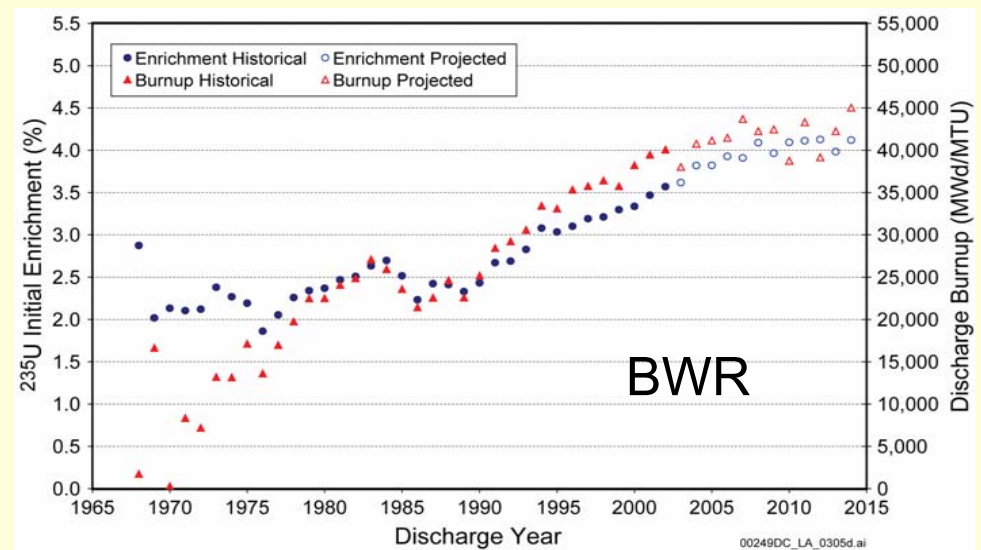
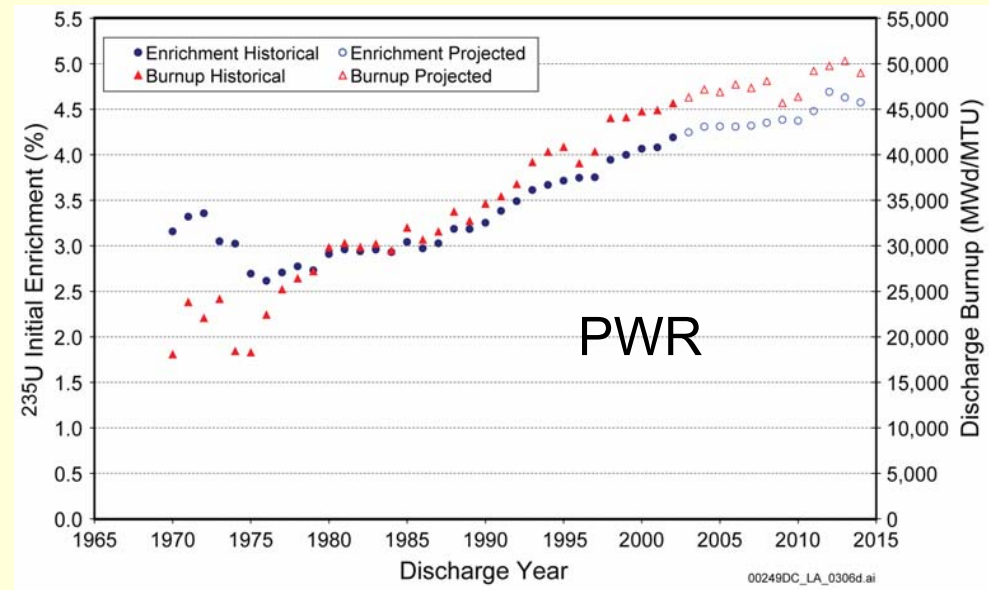


# Effects of Neutron Irradiation

- Elemental and isotopic composition changes
  - Actinides fission to produce energy
    - Rule-of-thumb: Fissioning 1% of actinides = 10 GWd/MT
    - Fission Products produced in amounts equal to actinides destroyed
  - Irradiation produces neutron capture products
    - Actinides: U-236, Np, Pu, Am, Cm, U-233
    - Hardware and fuel matrix: Activation products
      - Major constituents yield  $^{93}\text{Zr}$ ,  $^{60}\text{Co}$ , etc.
      - Important trace contaminants: U (transuranics), Li ( $^3\text{H}$ ), N ( $^{14}\text{C}$ )
- Physical changes: fuel swelling/cracking, clad embrittlement, fission gas release to plenum

# How much burnup?

- PWRs/BWRs
  - Now 40-50 GWd/MTHM
  - Climbing but enrichment challenges
- Fast reactors
  - Hope for 100+ GWd/MTHM
- Graphite-fueled reactors
  - Hope for 100+ GWd/MTHM



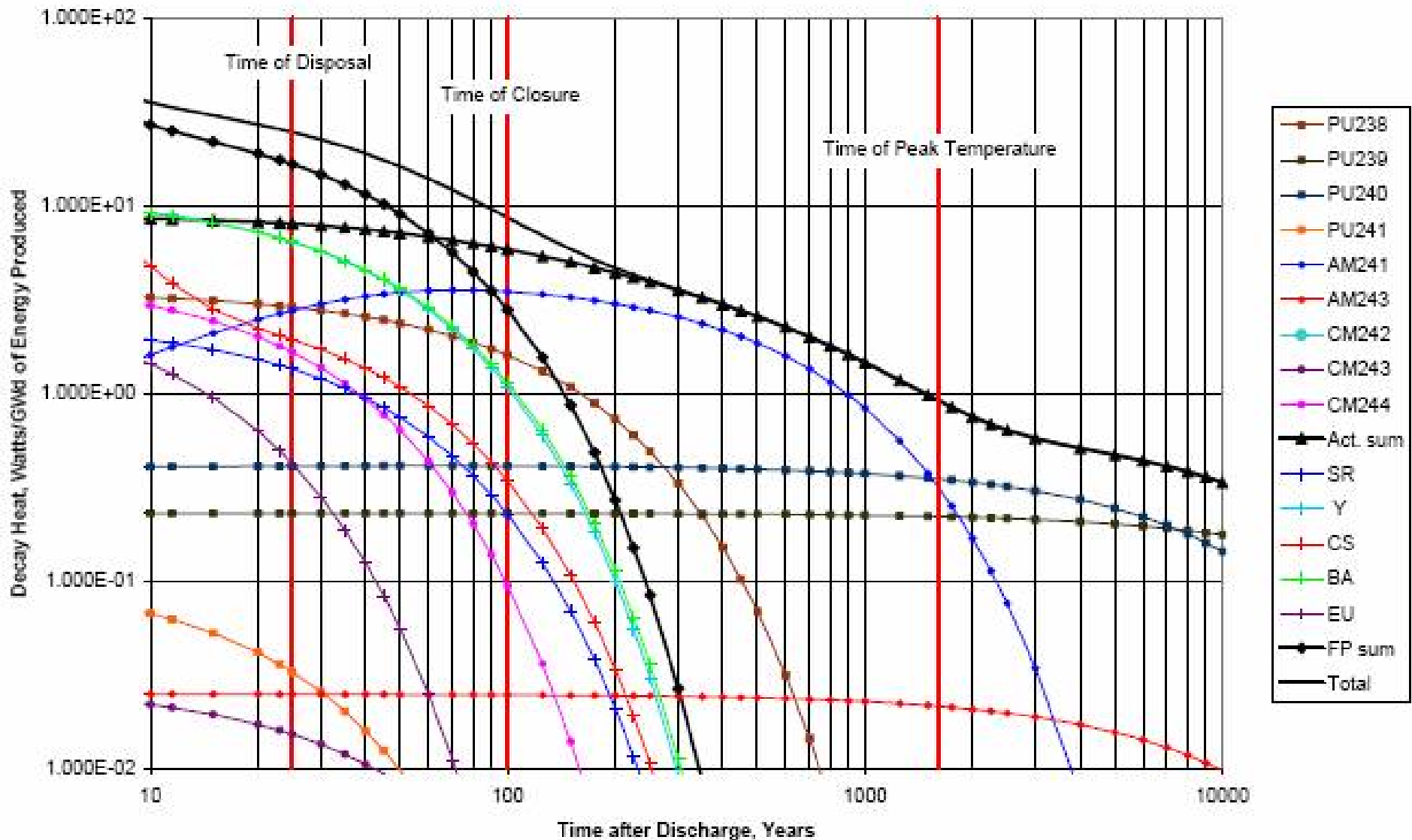


# What are the impacts?

- Complex mix of elements to be considered in separations (~entire periodic table)
- Greatly increased radioactivity
  - Gamma-rays/x-rays
  - Neutrons: spontaneous fission and ( $\alpha$ ,n)
  - Alpha (helium nucleus)
  - Beta (electron)
- Impacts
  - Penetrating radiation → need radiation shielding
  - Particles → material damage
  - Decay heat → provisions for heat removal



# Decay Heat, PWR, 33 GWd/MT

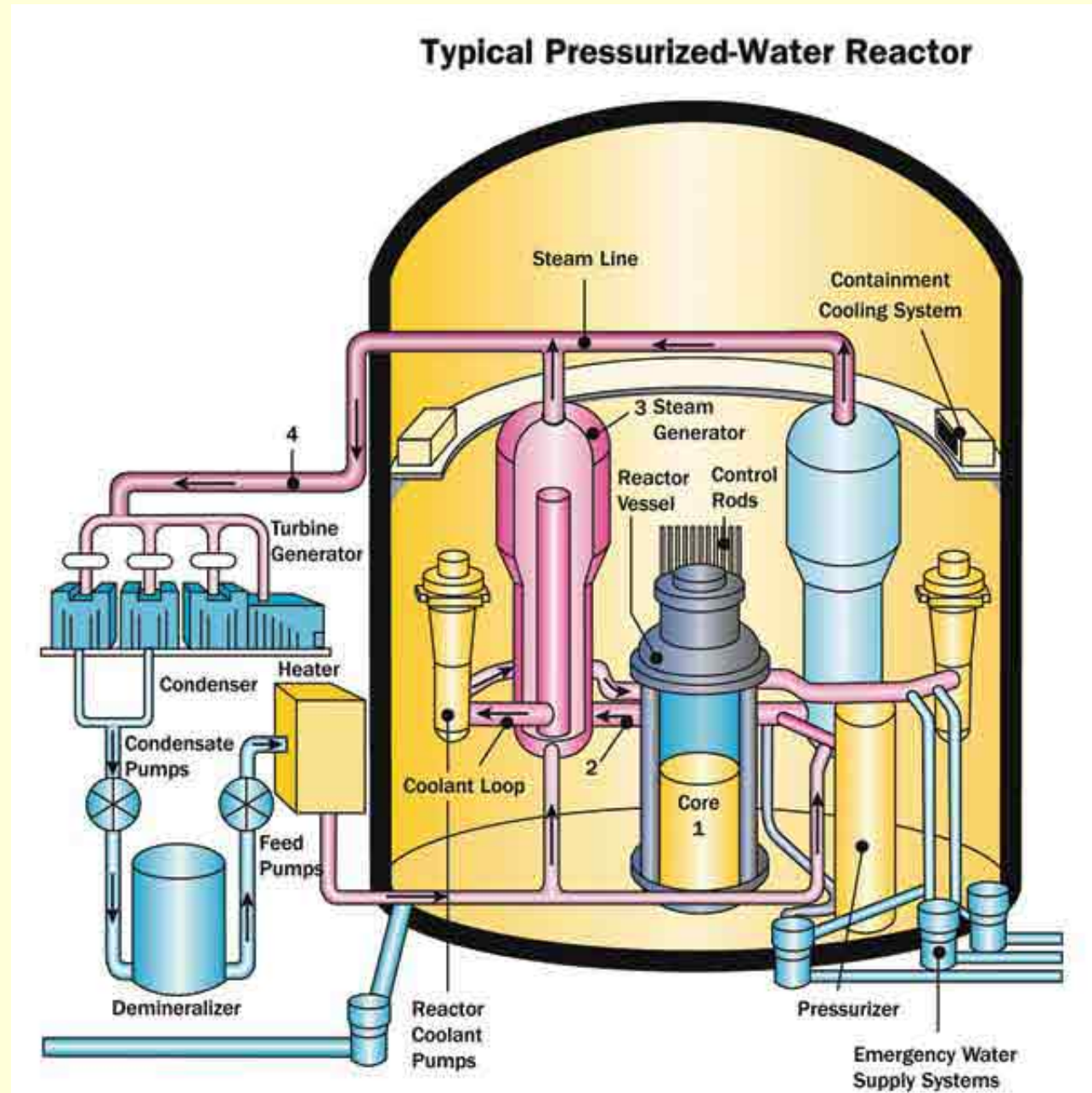




# Backup Slides

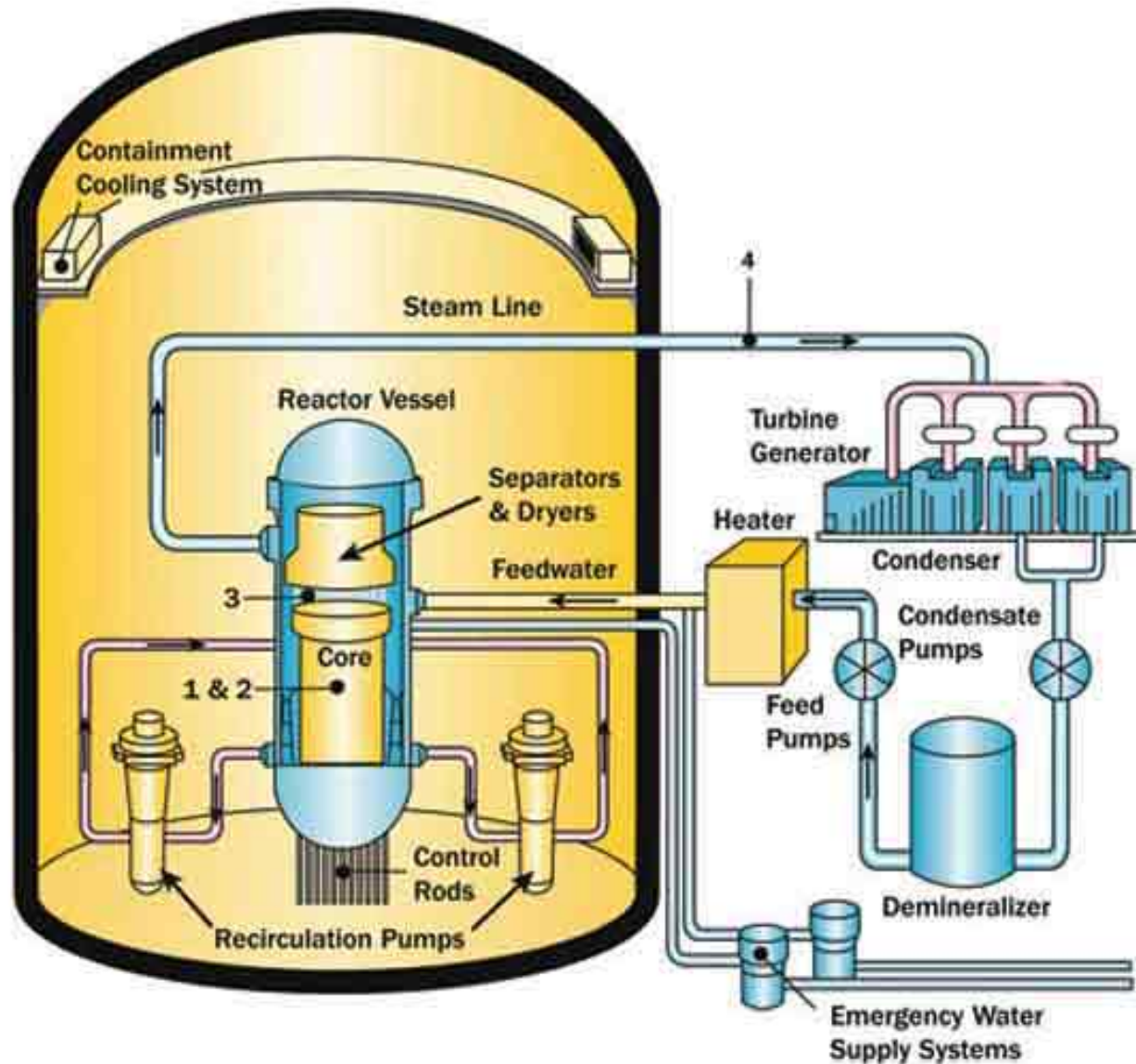


# Pressurized Water Reactor





# Boiling Water Reactor

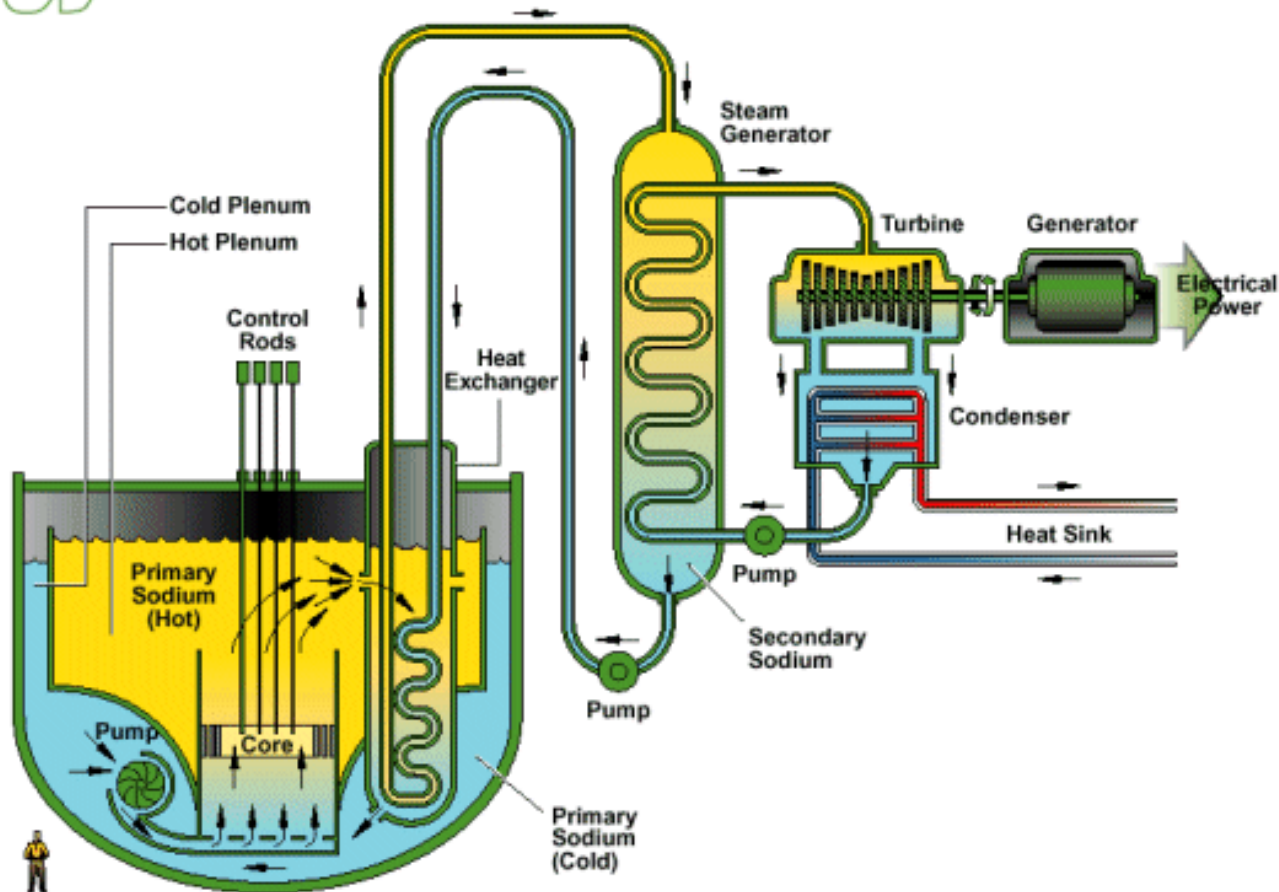




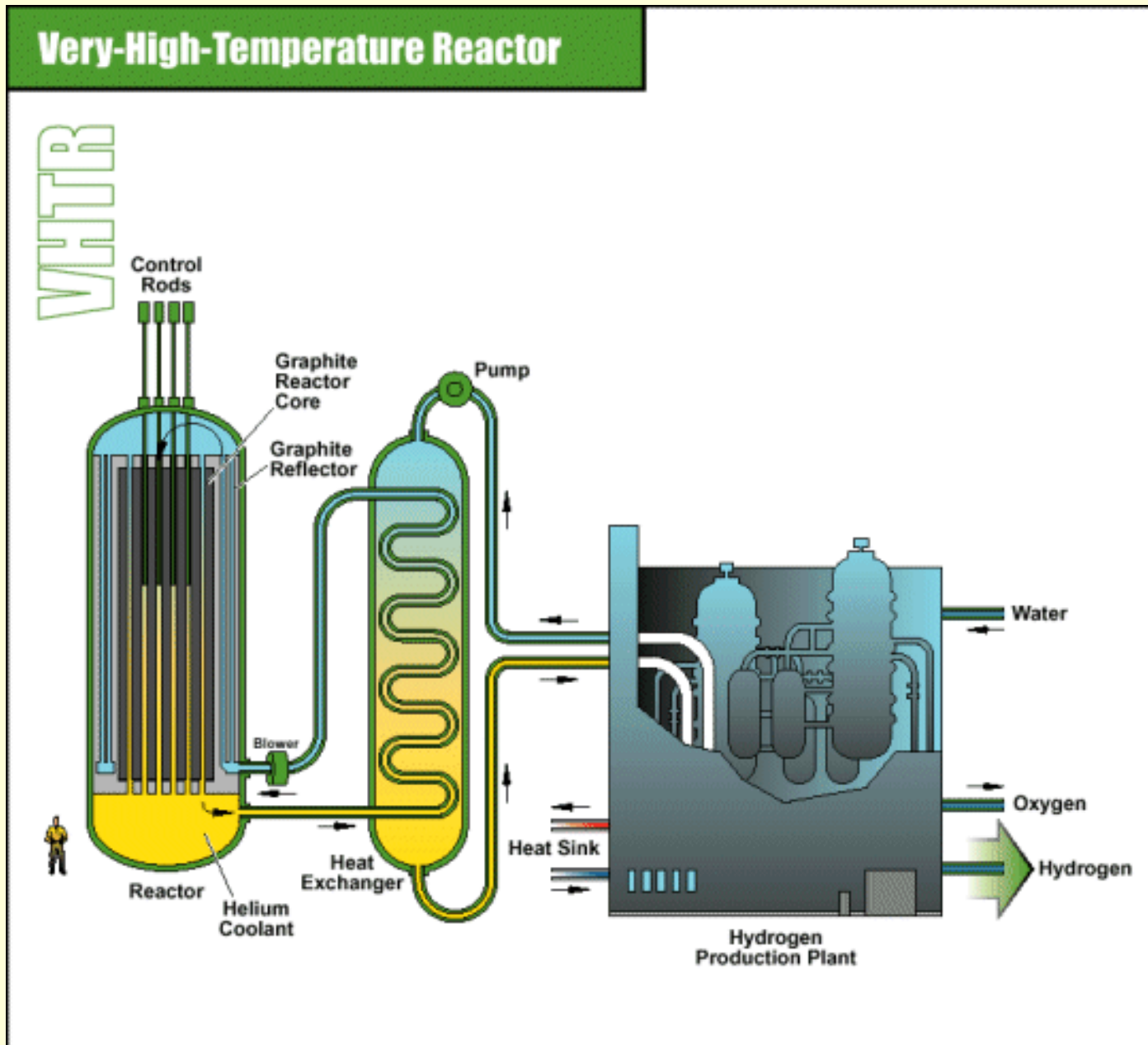
# Sodium-Cooled Fast Reactor

## Sodium-Cooled-Fast Reactor

SFR



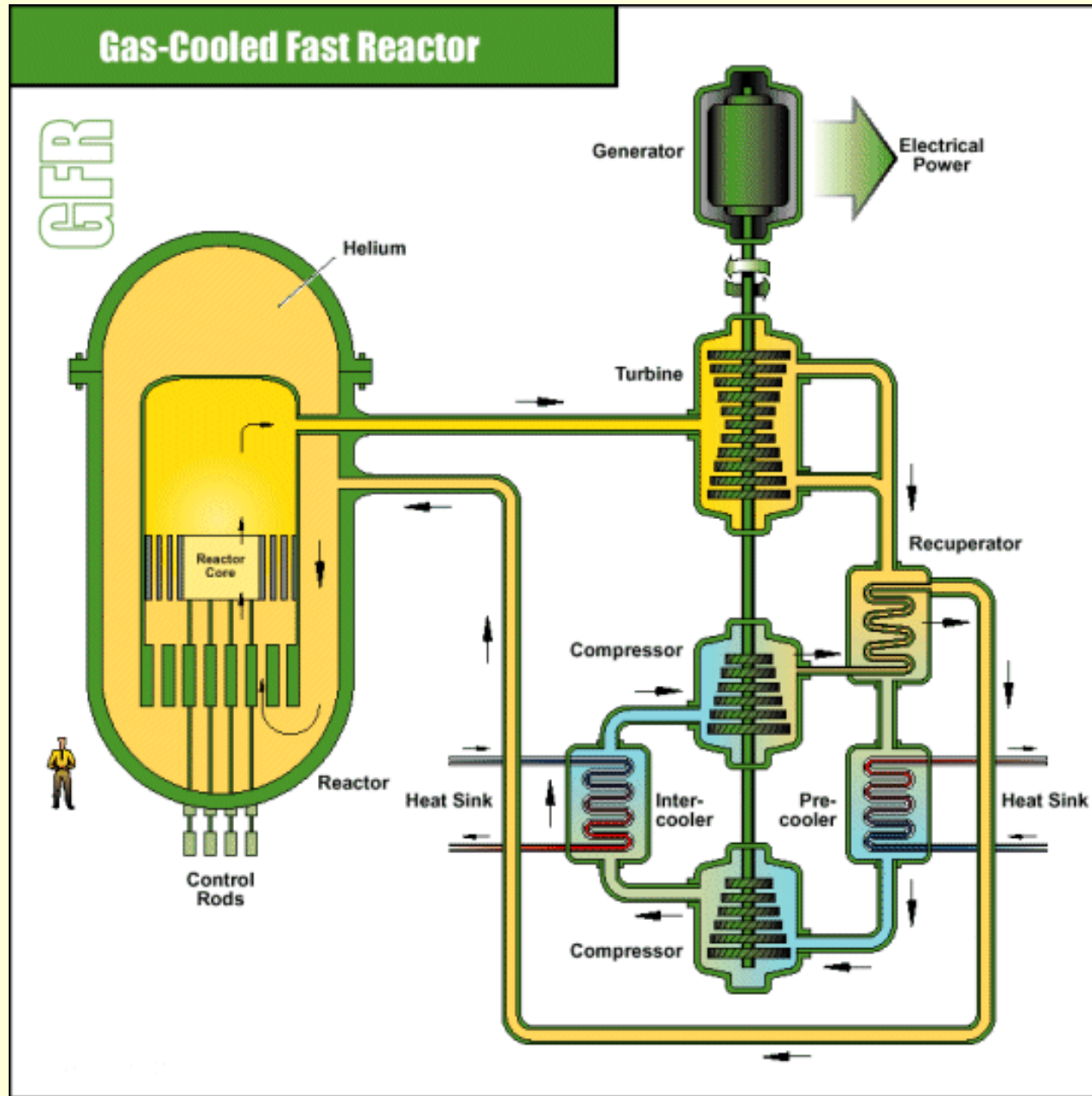
# Very-High-Temperature Reactor





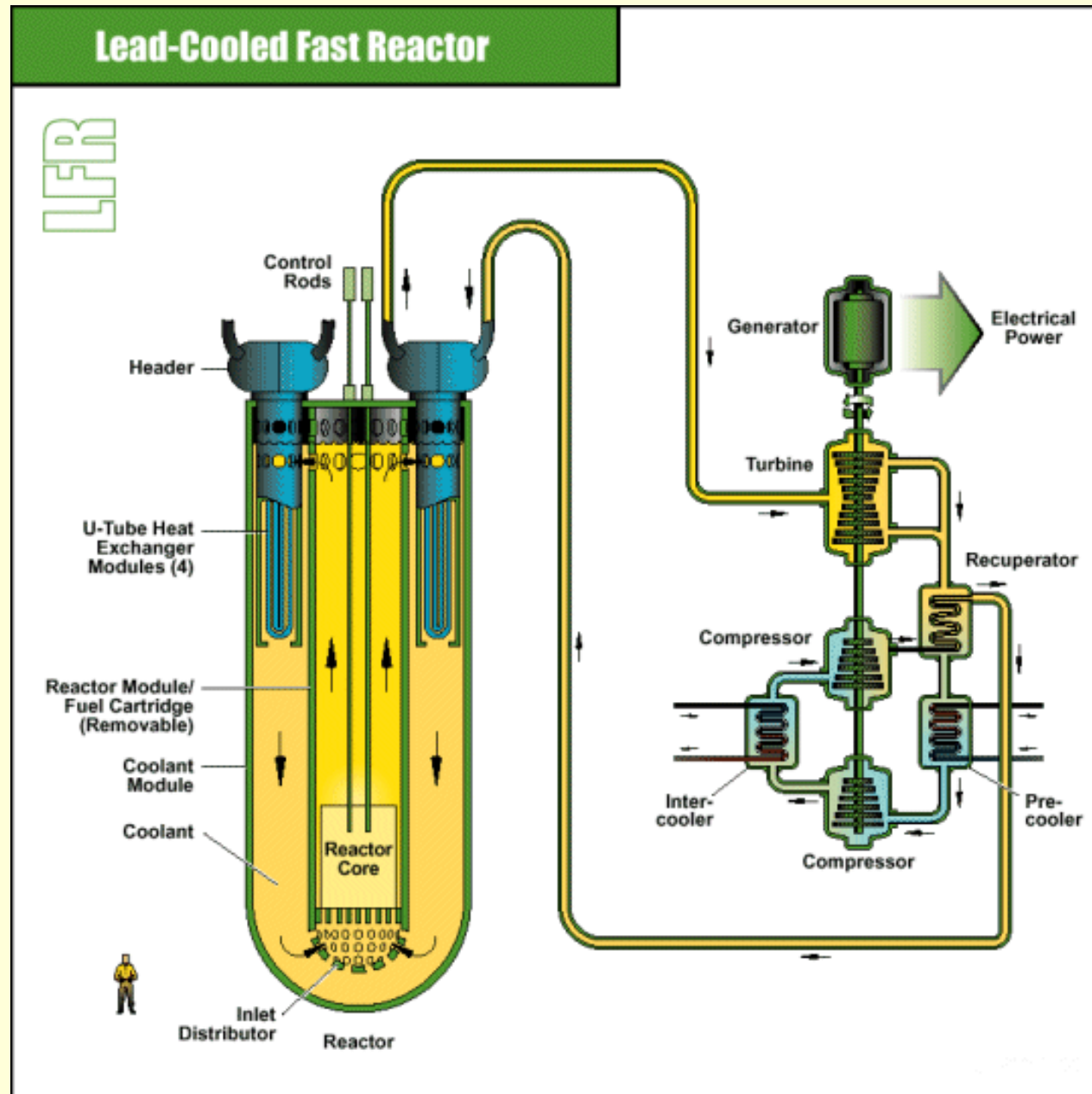


# Gas-Cooled Fast Reactor

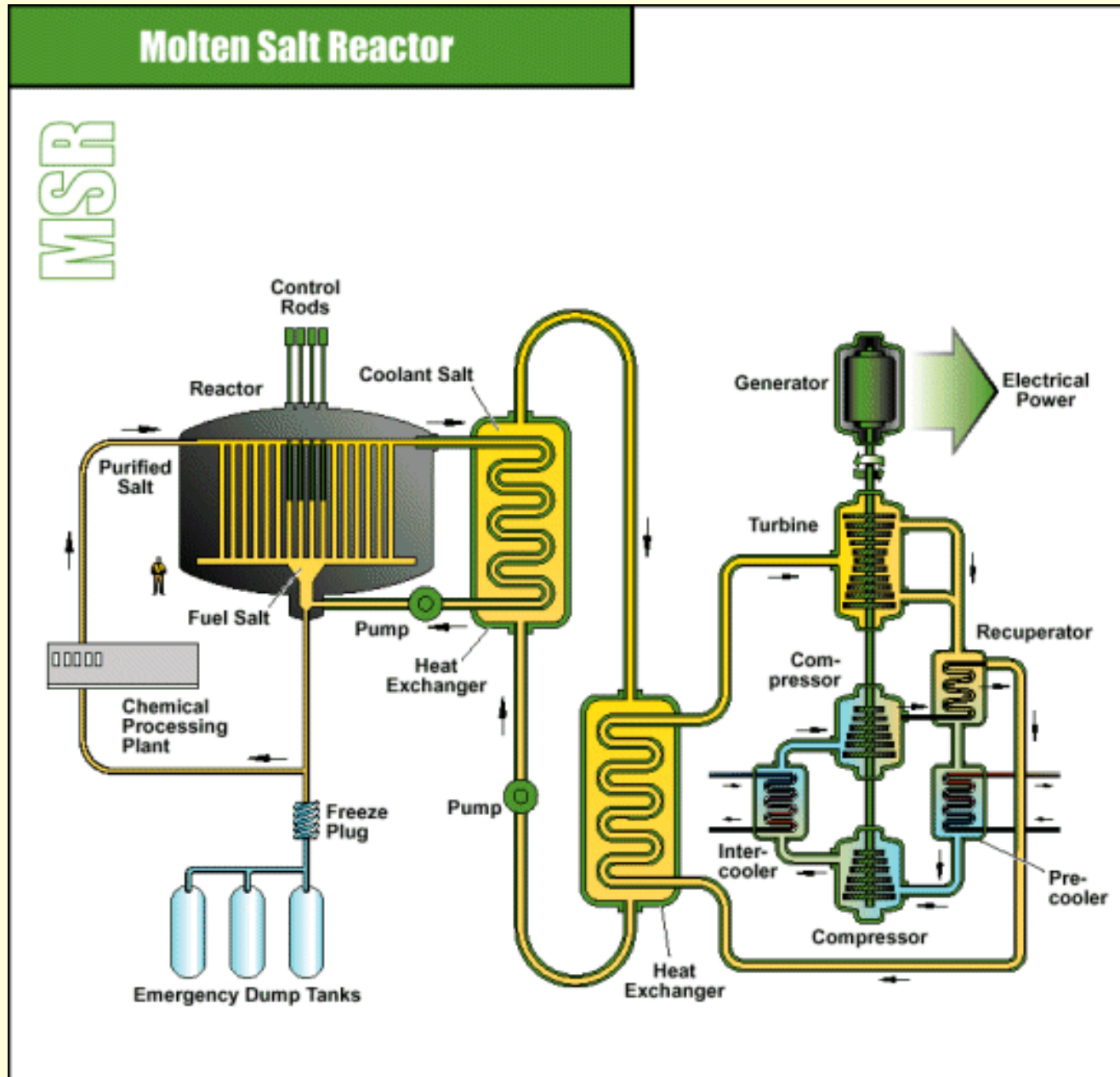




# Lead-Cooled Fast Reactor



# Molten Salt Reactor



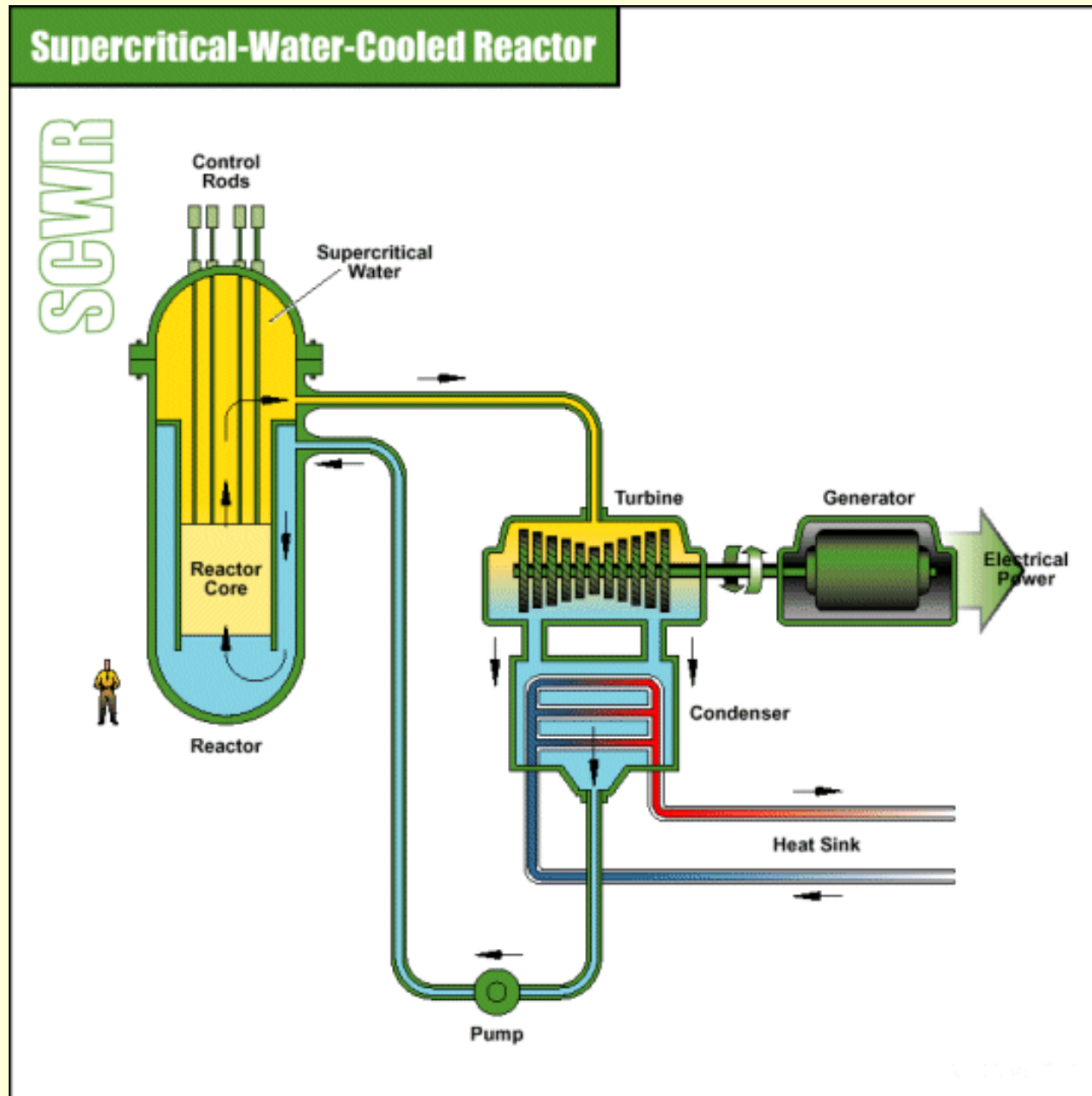




# Molten Salt Reactor Fuel

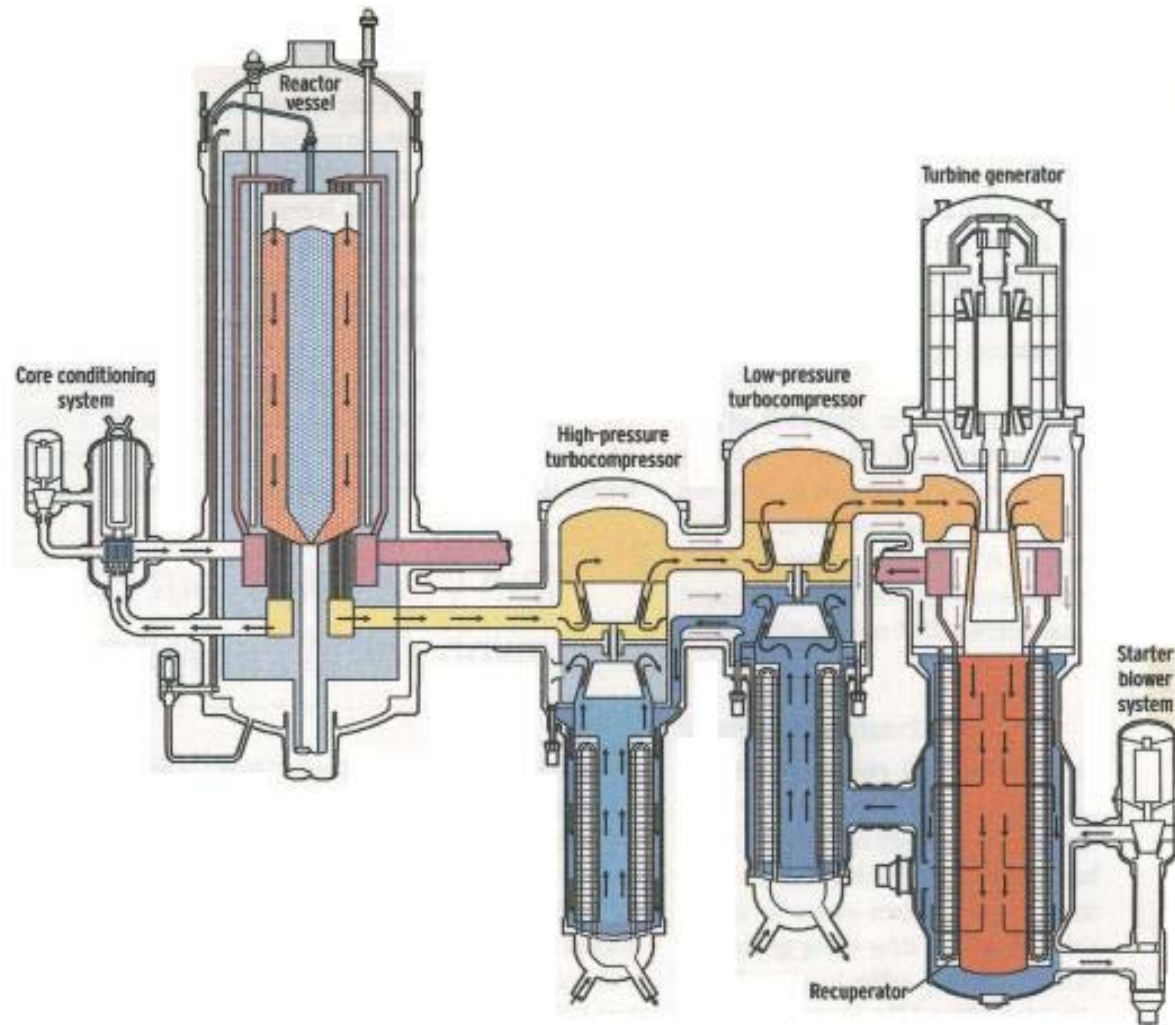


# Supercritical-Water-Cooled Reactor

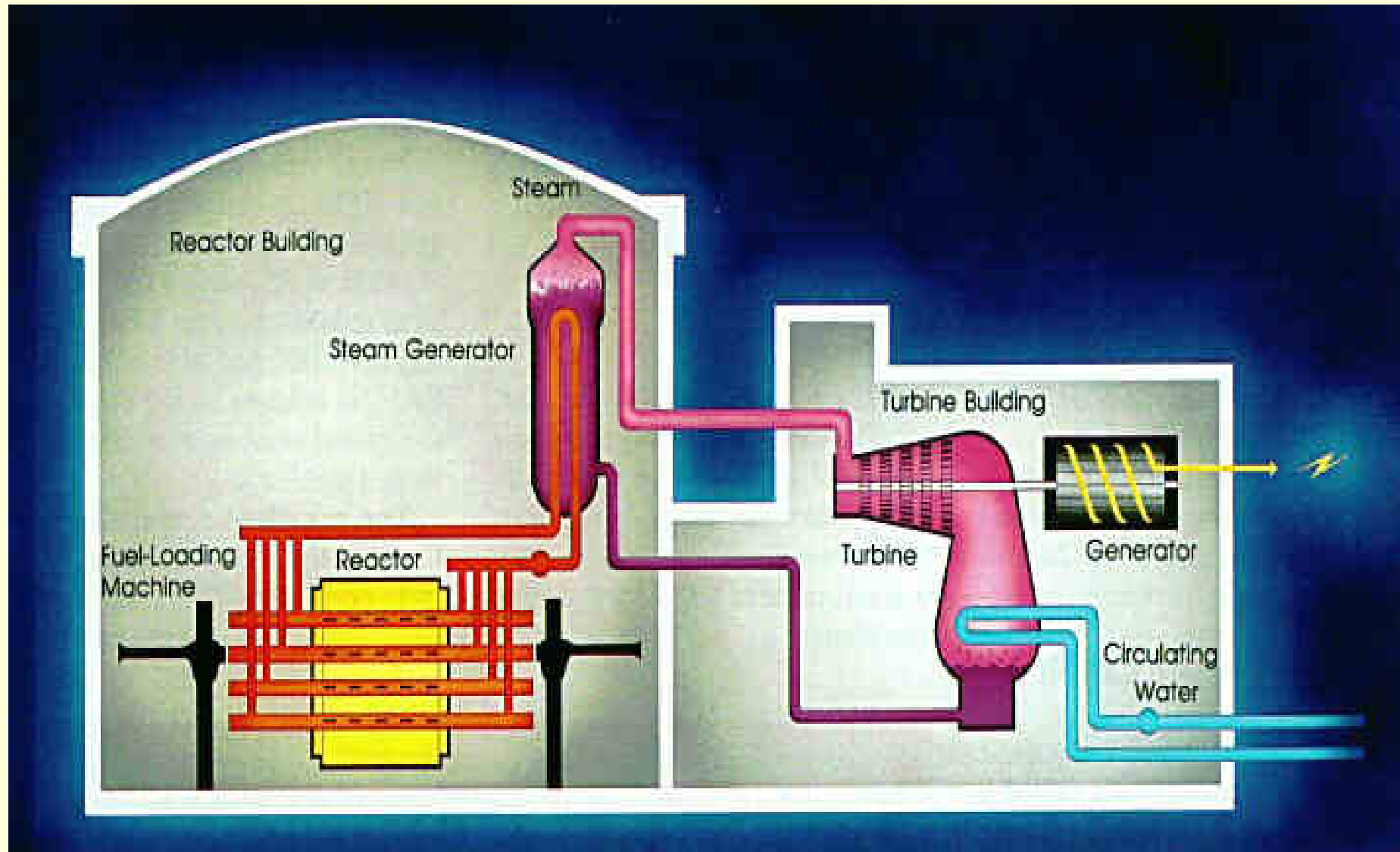




# Pebble-Bed Modular Reactor

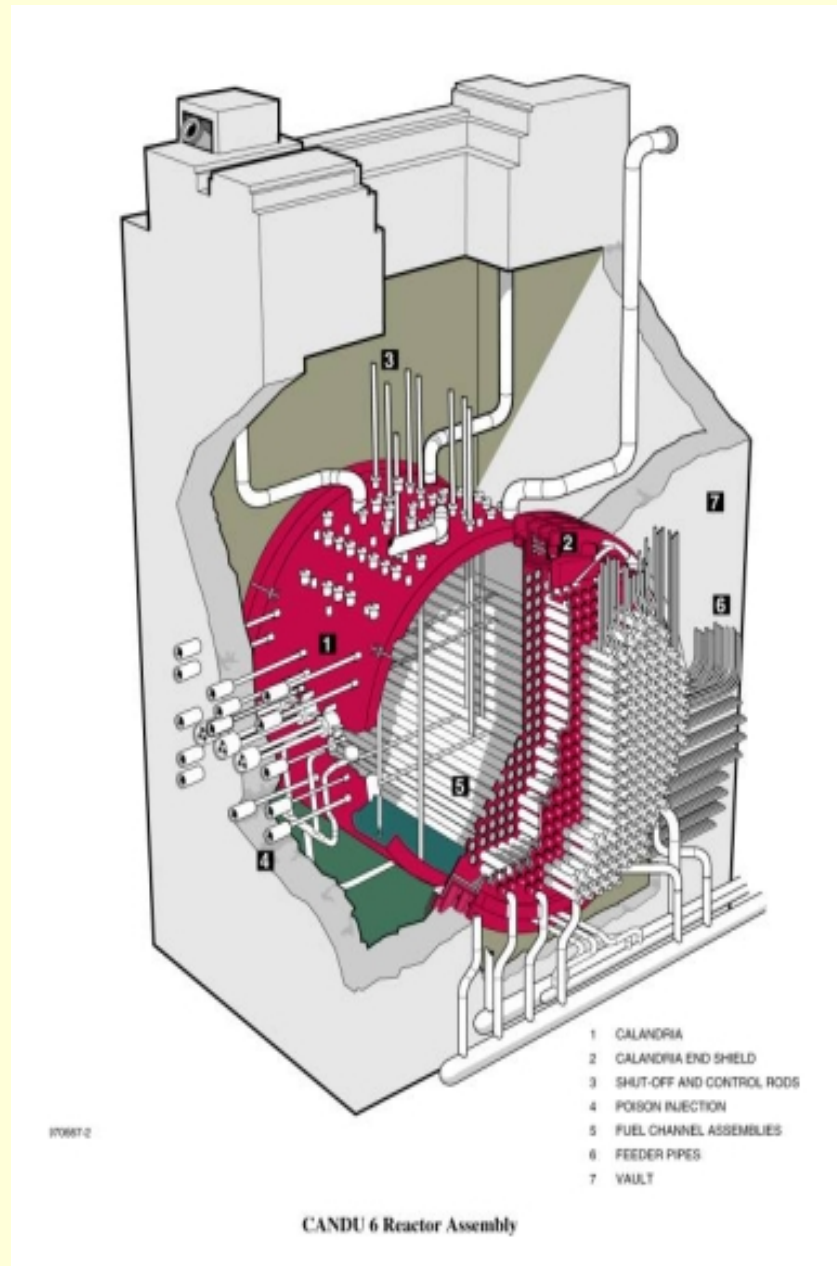


# Canada Deuterium-Uranium Reactor





# CANDU Calandria Schematic







# CANDU Calandria Photo





# CANDU Fuel Assembly



# CANDU Online Refueling

